Yong Hoon Jeong

List of Publications by Year
in descending order

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62 papers	1,354 citations	331670 21 h-index	361022 35 g-index
64 all docs	64 docs citations	64 times ranked	781 citing authors

#	Article	IF	Citations
1	Wettability of heated surfaces under pool boiling using surfactant solutions and nano-fluids. International Journal of Heat and Mass Transfer, 2008, 51, 3025-3031.	4.8	124
2	Subcooled flow boiling CHF enhancement with porous surface coatings. International Journal of Heat and Mass Transfer, 2007, 50, 3649-3657.	4.8	95
3	An experimental study on CHF enhancement in flow boiling using Al2O3 nano-fluid. International Journal of Heat and Mass Transfer, 2010, 53, 1015-1022.	4.8	95
4	Potential improvements of supercritical recompression CO2 Brayton cycle by mixing other gases for power conversion system of a SFR. Nuclear Engineering and Design, 2011, 241, 2128-2137.	1.7	91
5	Performance of supercritical Brayton cycle using CO2-based binary mixture at varying critical points for SFR applications. Nuclear Engineering and Design, 2013, 262, 12-20.	1.7	54
6	Critical Heat Flux Experiments on the Reactor Vessel Wall Using 2-D Slice Test Section. Nuclear Technology, 2005, 152, 162-169.	1.2	51
7	Experimental study on the pool boiling CHF enhancement using magnetite-water nanofluids. International Journal of Heat and Mass Transfer, 2012, 55, 2656-2663.	4.8	51
8	The effect of pressure on the critical heat flux in water-based nanofluids containing Al2O3 and Fe3O4 nanoparticles. International Journal of Heat and Mass Transfer, 2013, 61, 432-438.	4.8	51
9	Flow boiling critical heat flux characteristics of magnetic nanofluid at atmospheric pressure and low mass flux conditions. International Journal of Heat and Mass Transfer, 2013, 56, 101-106.	4.8	50
10	Critical heat flux for SiC- and Cr-coated plates under atmospheric condition. Annals of Nuclear Energy, 2015, 76, 335-342.	1.8	48
11	Flow boiling CHF enhancement with surfactant solutions under atmospheric pressure. International Journal of Heat and Mass Transfer, 2008, 51, 1913-1919.	4.8	38
12	Effect of heater material and coolant additives on CHF for a downward facing curved surface. Nuclear Engineering and Design, 2014, 278, 344-351.	1.7	38
13	Development of analytical model for solar chimney power plant with and without water storage system. Energy, 2016, 112, 200-207.	8.8	35
14	Safety evaluation of supercritical CO2 cooled micro modular reactor. Annals of Nuclear Energy, 2017, 110, 1202-1216.	1.8	35
15	CHF enhancement in flow boiling system with TSP and boric acid solutions under atmospheric pressure. Nuclear Engineering and Design, 2010, 240, 3594-3600.	1.7	32
16	A concept design of supercritical CO₂cooled SMR operating at isolated microgrid region. International Journal of Energy Research, 2017, 41, 512-525.	4.5	30
17	The effect of the geometric scale on the critical heat flux for the top of the reactor vessel lower head. Nuclear Engineering and Design, 2013, 258, 176-183.	1.7	28
18	Size effect of nanometer vacuum gap thermionic power conversion device with CsI coated graphite electrodes. Applied Physics Letters, 2009, 95, .	3.3	25

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19	Effects of two-phase flow conditions on flow boiling CHF enhancement of magnetite-water nanofluids. International Journal of Heat and Mass Transfer, 2014, 74, 278-284.	4.8	25
20	Innovative concept for an ultra-small nuclear thermal rocket utilizing a new moderated reactor. Nuclear Engineering and Technology, 2015, 47, 678-699.	2.3	24
21	Critical heat flux on downward-facing carbon steel flat plates under atmospheric condition. Experimental Thermal and Fluid Science, 2018, 90, 22-27.	2.7	23
22	Analysis of CHF enhancement by magnetite nanoparticle deposition in the subcooled flow boiling region. International Journal of Heat and Mass Transfer, 2017, 109, 1191-1199.	4.8	22
23	The effect of nanofluid stability on critical heat flux using magnetite-water nanofluids. Nuclear Engineering and Design, 2015, 292, 187-192.	1.7	21
24	Onset of Nucleate Boiling in narrow, rectangular channel for downward flow under low pressure. Annals of Nuclear Energy, 2017, 109, 498-506.	1.8	20
25	An investigation of pressure build-up effects due to check valveâ€™s closing characteristics using dynamic mesh techniques of CFD. Annals of Nuclear Energy, 2021, 152, 107996.	1.8	20
26	Investigation of the pressure vessel lower head potential failure under IVR-ERVC condition during a severe accident scenario in APR1400 reactors. Nuclear Engineering and Design, 2021, 376, 111107.	1.7	17
27	Conceptual design of reactor system for hybrid micro modular reactor (H-MMR) using potassium heat pipe. Nuclear Engineering and Design, 2020, 370, 110886.	1.7	16
28	Critical heat flux model on a downward facing surface for application to the IVR conditions. Nuclear Engineering and Design, 2018, 330, 317-324.	1.7	13
29	Effects of silica nanoparticles and low concentration on the deterioration of critical heat flux in a pool boiling experiment with a flat-type heater. International Journal of Heat and Mass Transfer, 2019, 144, 118420.	4.8	13
30	Feasibility study of solar-nuclear hybrid system for distributed power source. Energy Conversion and Management, 2021, 230, 113808.	9.2	13
31	Effect of dimensions and downward-facing angle on CHF under atmospheric condition. Experimental Thermal and Fluid Science, 2019, 102, 603-610.	2.7	11
32	Preliminary conceptual design of a new moderated reactor utilizing an LEU fuel for space nuclear thermal propulsion. Progress in Nuclear Energy, 2016, 91, 183-207.	2.9	10
33	Flow boiling CHF enhancement by wettability and flow conditions in a slug flow in the rectangular curved channel. Experimental Thermal and Fluid Science, 2018, 91, 388-398.	2.7	10
34	CHF experiment with downward-facing carbon and stainless steel plates under pressurized conditions. International Journal of Heat and Mass Transfer, 2018, 125, 670-680.	4.8	10
35	Simulation of the market penetration of hydrogen fuel cell vehicles in Korea. International Journal of Energy Research, 2008, 32, 318-327.	4.5	9
36	A heat transfer model development for CHF prediction with consideration of dry patch characteristics. International Journal of Heat and Mass Transfer, 2020, 148, 118812.	4.8	9

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37	A study on prediction methods of the critical heat flux for upward flow in a vertical narrow rectangular channel. Nuclear Engineering and Design, 2015, 294, 103-116.	1.7	8
38	Prediction of the critical heat flux for saturated upward flow boiling water in vertical narrow rectangular channels. Nuclear Engineering and Design, 2016, 303, 1-16.	1.7	8
39	CHF correlation development under ERVC conditions by using the local liquid velocity from PIV measurements. International Journal of Heat and Mass Transfer, 2019, 128, 171-184.	4.8	8
40	Flow boiling experiments for CHF evaluation under downward-facing heating including flow visualization: Effects of pressure, orientation, mass flux, and local quality. Annals of Nuclear Energy, 2022, 171, 108994.	1.8	8
41	Boric acid and boiling time effects on critical heat flux for corrosive and non-corrosive materials. Annals of Nuclear Energy, 2020, 136, 106999.	1.8	7
42	Fluid-to-fluid scaling criteria development for modeling high pressure steam condensation in horizontal tubes. International Journal of Heat and Mass Transfer, 2018, 116, 1266-1281.	4.8	5
43	The importance of representative aerosol diameter and bubble size distribution in pool scrubbing. Annals of Nuclear Energy, 2020, 147, 107712.	1.8	5
44	A novel approach for critical heat flux enhancement during severe accident mitigation with removal of radioactive materials from the coolant. Nuclear Engineering and Design, 2020, 365, 110715.	1.7	5
45	Mechanistic CHF model development for subcooled flow boiling in a vertical rectangular channel under low pressure. International Journal of Heat and Mass Transfer, 2021, 174, 121328.	4.8	5
46	Conceptual design of small modular reactor driven by natural circulation and study of design characteristics using CFD & RELAP5 code. Nuclear Engineering and Technology, 2020, 52, 2743-2759.	2.3	5
47	Experimental study of the nozzle size effect on aerosol removal by pool scrubbing. Nuclear Engineering and Design, 2021, 385, 111544.	1.7	5
48	CHF on Anodized Zirconium-based Alloy Surfaces with Protective Oxide Layers for ATF cladding. International Journal of Heat and Mass Transfer, 2021, 170, 120936.	4.8	4
49	Sodium-cooled fast reactor (SFR) fuel assembly design with graphite-moderating rods to reduce the sodium void reactivity coefficient. Nuclear Engineering and Design, 2014, 280, 223-232.	1.7	3
50	Heat transfer performance of downward-facing carbon and stainless steel surfaces. International Communications in Heat and Mass Transfer, 2020, 113, 104503.	5.6	3
51	Indirect validation of fluid-to-fluid scaling criteria for modeling steam condensation based on empirical correlations and CFD simulations. Nuclear Engineering and Design, 2021, 378, 111235.	1.7	3
52	Observation of the jet transition at a single vertical nozzle under pool scrubbing conditions. Annals of Nuclear Energy, 2022, 171, 109041.	1.8	3
53	Experimental evaluations of the critical heat flux in terms of the heater dimensions, orientation, and surface morphology. International Communications in Heat and Mass Transfer, 2022, 136, 106211.	5.6	3
54	Assessment of advanced RANS models ability to predict a turbulent swept liquid metal flow over a wire in a channel. Nuclear Engineering and Design, 2019, 353, 110206.	1.7	2

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55	An experimental observation of the effects of submicron- and micron-sized mesoporous silica particles on the critical heat flux. International Journal of Heat and Mass Transfer, 2020, 160, 120182.	4.8	2
56	Correction factor development for the 2006 Groeneveld CHF look-up table for rectangular channels under low pressure. Nuclear Engineering and Design, 2020, 370, 110869.	1.7	2
57	Analysis of CHF enhancement by magnetite nanoparticle deposition in the subcooled flow boiling region. , 2017, 109, 1191-1191.		1
58	Safety Enhancements for PHWRs Based on Macroscopic Losses of the Fukushima Accident. Science and Technology of Nuclear Installations, 2015, 2015, 1-12.	0.8	0
59	Experiment of CHF Enhancement by Magnetite Nanoparticle Deposition in the Subcooled Flow Boiling Region. , 2017, , .		0
60	Investigation of various reactor vessel auxiliary cooling system geometries for a hybrid micro modular reactor. Nuclear Engineering and Design, 2021, 379, 111239.	1.7	0
61	Depressurization of nuclear power plants through a silica gel-based system. Nuclear Engineering and Design, 2021, 381, 111333.	1.7	0
62	Magnetic withdrawal of particles for multiple purposes in nuclear power plants. Nuclear Engineering and Technology, 2021, 53, 3979-3989.	2.3	0