

Jeong-Yong Park

List of Publications by Year in descending order

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papers

669
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567281

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43
all docs

43
docs citations

43
times ranked

531
citing authors

#	ARTICLE	IF	CITATIONS
1	Remote injection casting process with reduced material loss for fabrication of metallic fuels. Progress in Nuclear Energy, 2021, 132, 103595.	2.9	4
2	Potential source of carbon contamination in U-Zr melt processed in Y2O3-plasma-spray-coated graphite. Journal of Nuclear Materials, 2021, 547, 152810.	2.7	3
3	Preliminary study of fabrication technology for transmutation fuels using surrogate alloy. Journal of Radioanalytical and Nuclear Chemistry, 2020, 324, 451-457.	1.5	0
4	Ceramic plasma-spray-coated graphite crucible for injection casting of fast reactor fuel slugs. International Journal of Applied Ceramic Technology, 2018, 15, 991-998.	2.1	1
5	Preparation of metallic fuel rodlets for irradiation testing in the HANARO research reactor. Journal of Radioanalytical and Nuclear Chemistry, 2018, 315, 137-143.	1.5	2
6	Characterization of reused U-10Zr heel residue containing rare-earth elements through surface treatment. Journal of Radioanalytical and Nuclear Chemistry, 2018, 316, 1157-1163.	1.5	3
7	Interaction studies between U-Zr alloy system and ceramic plasma-spray coated layer at elevated temperature. Progress in Nuclear Science and Technology, 2018, 5, 192-195.	0.3	0
8	Phase characteristics of rare earth elements in metallic fuel for a sodium-cooled fast reactor by injection casting. Journal of Nuclear Materials, 2017, 486, 53-59.	2.7	11
9	Quantitative and isotopic analysis of released and retained krypton and xenon fission gases from irradiated metallic fuels. Journal of Radioanalytical and Nuclear Chemistry, 2017, 312, 517-521.	1.5	6
10	Injection casting of U-Zr and U-Zr-RE fuel slugs and their characterization. Journal of Nuclear Science and Technology, 2017, 54, 648-654.	1.3	5
11	Crystallographic characterization of irradiated U-10Zr and U-10Zr-5Ce metallic fuels. Journal of Radioanalytical and Nuclear Chemistry, 2017, 314, 2579-2583.	1.5	1
12	Analysis of SiC-Layer for Graphite Crucible Coating for Casting of Metal Fuel Slugs for Sodium-Cooled Fast Reactor. Journal of Nanoscience and Nanotechnology, 2017, 17, 8587-8592.	0.9	2
13	Characterization of Plasma-Sprayed Ceramic Coating Layers for Melting Crucible of Metal Fuels. Journal of Nanoscience and Nanotechnology, 2017, 17, 8598-8602.	0.9	0
14	Effect of TiSi ₂ /Ti ₃ SiC ₂ matrix phases in a reaction-bonded SiC on mechanical and high-temperature oxidation properties. Journal of the European Ceramic Society, 2016, 36, 1343-1348.	5.7	15
15	Metal Fuel Development and Verification for Prototype Generation IV Sodium-Cooled Fast Reactor. Nuclear Engineering and Technology, 2016, 48, 1096-1108.	2.3	22
16	Irradiation-induced disordering and amorphization of Al ₃ Ti-based intermetallic compounds. Journal of Nuclear Materials, 2015, 467, 601-606.	2.7	7
17	Experimental investigation on the corrosion behavior of Al ₃ Ti-based intermetallic compounds in nuclear reactor normal operation conditions. Journal of Nuclear Materials, 2015, 467, 607-611.	2.7	4
18	Effect of preceramic and Zr coating on impregnation behaviors of SiC ceramic composite. Metals and Materials International, 2015, 21, 173-178.	3.4	3

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19	Effect of dissolved hydrogen on the corrosion behavior of chemically vapor deposited SiC in a simulated pressurized water reactor environment. <i>Corrosion Science</i> , 2015, 98, 304-309.	6.6	47
20	Microstructures of laser bonded SiC ceramics with Zr interlayers. <i>Journal of Nuclear Materials</i> , 2014, 455, 586-590.	2.7	8
21	In situ heating TEM analysis of oxide layer formed on Zr-1.5Nb alloy. <i>Journal of Nuclear Materials</i> , 2014, 451, 189-197.	2.7	8
22	KAERI's Development of LWR Accident-Tolerant Fuel. <i>Nuclear Technology</i> , 2014, 186, 295-304.	1.2	68
23	Microstructures of diffusion bonded SiC ceramics using Ti and Mo interlayers. <i>Journal of Nuclear Materials</i> , 2013, 441, 510-513.	2.7	36
24	Long-term corrosion behavior of CVD SiC in 360°C water and 400°C steam. <i>Journal of Nuclear Materials</i> , 2013, 443, 603-607.	2.7	56
25	High temperature steam oxidation of Al3Ti-based alloys for the oxidation-resistant surface layer on Zr fuel claddings. <i>Journal of Nuclear Materials</i> , 2013, 437, 75-80.	2.7	22
26	Effect of Al ₂ O ₃ Coating as a Corrosion Barrier of Ferritic-Martensitic Steel in Pb-Li Melt. <i>Fusion Science and Technology</i> , 2013, 64, 221-224.	1.1	6
27	High Heat Flux Test of the KO Standard Mockups for ITER First Wall Semi-Prototype. <i>Fusion Science and Technology</i> , 2011, 60, 161-164.	1.1	1
28	Joining of Be to Ferritic-Martensitic Steels with Diffusion Barrier Interlayer. <i>Fusion Science and Technology</i> , 2011, 60, 422-425.	1.1	3
29	Investigation on the microstructure and mechanical properties of CuCrZr after manufacturing thermal cycle for plasma facing component. <i>Journal of Nuclear Materials</i> , 2011, 417, 916-919.	2.7	14
30	Modeling of thermal creep behavior in Zr-1.1Nb-0.05Cu alloys. <i>Metals and Materials International</i> , 2011, 17, 15-19.	3.4	2
31	HIP joining of RAFM/RAFM steel and beryllium/RAFM steel for fabrication of the ITER TBM first wall. <i>Metals and Materials International</i> , 2009, 15, 465-470.	3.4	17
32	Simple approach to the mechanical anisotropy of cold-rolled zirconium alloys. <i>Metals and Materials International</i> , 2009, 15, 803-807.	3.4	2
33	Fabrication of Be/CuCrZr/SS mock-ups for ITER first wall. <i>Fusion Engineering and Design</i> , 2009, 84, 1468-1471.	1.9	25
34	Effect of ion-beam assisted deposition on resistivity and crystallographic structure of Cr/Cu. <i>Electronic Materials Letters</i> , 2009, 5, 105-107.	2.2	4
35	Corrosion and oxide characteristics of Zr-1.5Nb-0.4Sn-0.2Fe-0.1Cr alloys in 360°C pure water and LiOH solution. <i>Journal of Nuclear Materials</i> , 2008, 373, 343-350.	2.7	27
36	Effect of cooling rate on mechanical properties of aged ITER-grade CuCrZr. <i>Fusion Engineering and Design</i> , 2008, 83, 1503-1507.	1.9	28

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37	Corrosion behavior and oxide properties of Zr~1.1wt%Nb~0.05wt%Cu alloy. Journal of Nuclear Materials, 2006, 359, 59-68.	2.7	53
38	Corrosion behavior of Zr~Nb alloys in 360~C water and 500~C supercritical water. Metals and Materials International, 2006, 12, 497-503.	3.4	1
39	Out-of-pile and In-pile Performance of Advanced Zirconium Alloys (HANA) for High Burn-up Fuel. Journal of Nuclear Science and Technology, 2006, 43, 977-983.	1.3	46
40	Out-of-pile and In-pile Performance of Advanced Zirconium Alloys (HANA) for High Burn-up Fuel. Journal of Nuclear Science and Technology, 2006, 43, 977-983.	1.3	16
41	Corrosion behavior of Zr alloys with a high Nb content. Journal of Nuclear Materials, 2005, 340, 237-246.	2.7	48
42	Crystal structure and grain size of Zr oxide characterized by synchrotron radiation microdiffraction. Journal of Nuclear Materials, 2004, 335, 433-442.	2.7	32
43	Effects of precipitation characteristics on the out-of-pile corrosion behavior of niobium-containing zirconium alloys. Metals and Materials International, 2001, 7, 447-455.	3.4	10