

# Otman Jai

## List of Publications by Year in descending order

Source: <https://exaly.com/author-pdf/1763684/publications.pdf>

Version: 2024-02-01

16  
papers

70  
citations

1684188

5  
h-index

1588992

8  
g-index

16  
all docs

16  
docs citations

16  
times ranked

30  
citing authors

#	ARTICLE	IF	CITATIONS
1	Neutron activation analysis: Modelling studies to improve the neutron flux of Americium-241 Beryllium source. Nuclear Engineering and Technology, 2017, 49, 787-791.	2.3	20
2	Monte Carlo simulation of thermal neutron flux of americium-241 beryllium source used in neutron activation analysis. Moscow University Physics Bulletin (English Translation of Vestnik Moskovskogo Universiteta, Fizika), 2017, 60, 107-110.	0.0	0
3	Modelisation and distribution of neutron flux in radium-226 beryllium source (226Ra-Be). Moscow University Physics Bulletin (English Translation of Vestnik Moskovskogo Universiteta, Fizika), 2017, 72, 465-469.	0.4	8
4	Validation of DRAGON4/DONJON4 simulation methodology for a typical MNSR by calculating reactivity feedback coefficient and neutron flux. Results in Physics, 2018, 9, 1155-1160.	4.1	7
5	Spallation Yield of Neutrons Produced in Tungsten and Bismuth Target Bombarded with 0.1 to 3 GeV Proton Beam. Moscow University Physics Bulletin (English Translation of Vestnik Moskovskogo Universiteta, Fizika), 2018, 61, 107-110.	0.784314	0
6	Validation Study of the Reactor Physics Lattice Transport Code DRAGON5 Based on DRAGLIB Libraries by TRX and BAPL Critical Experiments of Light Water Reactors for Neutronic Analysis of TRIGA MARK-II Research Reactor. Moscow University Physics Bulletin (English Translation of Vestnik Moskovskogo Universiteta, Fizika), 2018, 61, 107-110.	0.4	3
7	Investigation of spallation neutron production by 1 GeV protons beam. International Journal of Nuclear Energy Science and Technology, 2018, 12, 324.	0.0	3
8	Toward for production of Molybdenum-99 by irradiation of MoO3 target in a neutron flux. Bangladesh Journal of Medical Science, 2018, 17, 567-572.	0.2	2
9	Derivation of optimal process MOC parameters and analysis of the 2D C5G7 MOX benchmark using DRAGON5 code. Nuclear Engineering and Design, 2022, 388, 111613.	1.7	2
10	Monte Carlo transport code use for optimisation of neutron flux produced with 10-18 MeV electron beam energy. International Journal of Nuclear Energy Science and Technology, 2018, 12, 313.	0.0	1
11	Deterministic Analysis of the Low Enriched Uranium SLOWPOKE-2 Research Reactor Using DRAGON-5 and DONJON-5 Codes System. Moscow University Physics Bulletin (English Translation of Vestnik Moskovskogo Universiteta, Fizika), 2018, 61, 107-110.	0.784314	0
12	Modeling of the 2 MW TRIGA Mark-II Research Reactor Using the Deterministic Codes System DRAGON5 and DONJON5. Moscow University Physics Bulletin (English Translation of Vestnik Moskovskogo Universiteta, Fizika), 2018, 61, 107-110.	0.0	0
13	Monte Carlo transport code using for simulating the neutron yield of spallation targets: Uranium, Thorium, and Tantalum are used for an accelerator based on high proton beam. , 2018, , .		0
14	Investigation of spallation neutron production by 1 GeV protons beam. International Journal of Nuclear Energy Science and Technology, 2018, 12, 324.	0.0	0
15	Monte Carlo transport code use for optimisation of neutron flux produced with 10-18 MeV electron beam energy. International Journal of Nuclear Energy Science and Technology, 2018, 12, 313.	0.0	0
16	New study of various target neutron yields from spallation reactions using a high-energy proton beam. International Journal of Nuclear Energy Science and Technology, 2019, 13, 120.	0.0	0