

Ali Hainoun

List of Publications by Year in Descending Order

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The third column is the impact factor (IF) of the journal, and the fourth column is the number of citations of the article.

30 papers	324 citations	11 h-index	17 g-index
30 ext. papers	399 ext. citations	3.5 avg, IF	3.22 L-index

#	Paper	IF	Citations
30	A review of spatio-temporal urban energy system modeling for urban decarbonization strategy formulation. <i>Renewable and Sustainable Energy Reviews</i> , 2022 , 162, 112426	16.2	2
29	Simulation of subcooled flow instability for high flux research reactors with ATHLET. <i>Kerntechnik</i> , 2022 , 66, 214-216	0.4	
28	Analysis and Evaluation of the Feasibility of Positive Energy Districts in Selected Urban Typologies in Vienna Using a Bottom-Up District Energy Modelling Approach. <i>Energies</i> , 2021 , 14, 4449	3.1	4
27	Smarter Together: Progressing Smart Data Platforms in Lyon, Munich, and Vienna. <i>Energies</i> , 2021 , 14, 1075	3.1	0
26	Techno-economic optimisation of long-term energy supply strategy of Vienna city. <i>Energy Policy</i> , 2021 , 158, 112554	7.2	1
25	Long-term expansion planning for the Syrian electric system using the optimisation model WASP-IV. <i>International Journal of Global Energy Issues</i> , 2015 , 38, 164	0.3	1
24	Comparative safety assessment of HEU and LEU core of MNSR under large reactivity insertion accidents. <i>Annals of Nuclear Energy</i> , 2014 , 63, 624-632	1.7	2
23	International benchmark study of advanced thermal hydraulic safety analysis codes against measurements on IEA-R1 research reactor. <i>Nuclear Engineering and Design</i> , 2014 , 280, 233-250	1.8	18
22	Future development of Syrian power sector in view of GHG mitigation options. <i>Renewable and Sustainable Energy Reviews</i> , 2014 , 38, 1045-1055	16.2	3
21	A comparative assessment of independent thermal-hydraulic models for research reactors: The RSG-GAS case. <i>Nuclear Engineering and Design</i> , 2014 , 268, 77-86	1.8	8
20	Thermal hydraulic and safety analysis for core conversion (HEU→LEU) of Syrian Miniature Neutron Source Reactor. <i>Progress in Nuclear Energy</i> , 2012 , 60, 140-145	2.3	3
19	Thermal hydraulic analysis of Syrian MNSR research reactor using RELAP5/Mod3.2 code. <i>Annals of Nuclear Energy</i> , 2010 , 37, 572-581	1.7	17
18	Safety analysis of the IAEA reference research reactor during loss of flow accident using the code MERSAT. <i>Nuclear Engineering and Design</i> , 2010 , 240, 1132-1138	1.8	11
17	Formulating an optimal long-term energy supply strategy for Syria using MESSAGE model. <i>Energy Policy</i> , 2010 , 38, 1701-1714	7.2	48
16	Safety analysis of the reference research reactor MTR during reactivity insertion accident using the code MERSAT. <i>Annals of Nuclear Energy</i> , 2010 , 37, 853-860	1.7	5
15	Estimating the health damage costs of syrian electricity generation system using impact pathway approach. <i>Energy</i> , 2010 , 35, 628-638	7.9	36
14	Construction of the hourly load curves and detecting the annual peak load of future Syrian electric power demand using bottom-up approach. <i>International Journal of Electrical Power and Energy Systems</i> , 2009 , 31, 1-12	5.1	19

13	Determination of major kinetic parameters of the Syrian MNSR for different fuel loading using Monte Carlo technique. <i>Annals of Nuclear Energy</i> , 2009 , 36, 1663-1667	1.7	10
12	Core conversion analyses of the Syrian MNSR reactor from HEU to LEU and MEU fuel with homogeneously mixed burnable poisons. <i>Applied Radiation and Isotopes</i> , 2009 , 67, 1919-24	1.7	7
11	Modification and validation of the natural heat convection and subcooled void formation models in the code PARET. <i>Annals of Nuclear Energy</i> , 2008 , 35, 395-403	1.7	2
10	Simulation of LOFA and RIA for the IEA-R1 research reactor using the code MERSAT. <i>Annals of Nuclear Energy</i> , 2008 , 35, 2093-2104	1.7	9
9	Analysis of MNSR core composition changes using the codes WIMSD-4 and CITATION. <i>Applied Radiation and Isotopes</i> , 2008 , 66, 1492-500	1.7	8
8	Conceptual design modifications of the cooling system of MNSR reactor to increase its maximum continuous operation time. <i>Nuclear Engineering and Design</i> , 2007 , 237, 2275-2281	1.8	3
7	Analysis of the Syrian long-term energy and electricity demand projection using the end-use methodology. <i>Energy Policy</i> , 2006 , 34, 1958-1970	7.2	27
6	Full-scale modelling of the MNSR reactor to simulate normal operation, transients and reactivity insertion accidents under natural circulation conditions using the thermal hydraulic code ATHLET. <i>Nuclear Engineering and Design</i> , 2005 , 235, 33-52	1.8	18
5	Dynamic analysis of the closed-loop transfer function in the miniature neutron source reactor (MNSR). <i>Nuclear Engineering and Design</i> , 2004 , 232, 19-28	1.8	4
4	Measurement of the Syrian MNSR delayed neutron fraction and neutron generation time by noise analysis. <i>Annals of Nuclear Energy</i> , 2004 , 31, 331-341	1.7	3
3	Simulation of subcooled flow instability for high flux research reactors using the extended code ATHLET. <i>Nuclear Engineering and Design</i> , 2001 , 207, 163-180	1.8	11
2	Determination of neutron generation time in miniature neutron source reactor by measurement of neutronics transfer function. <i>Nuclear Engineering and Design</i> , 2000 , 195, 299-305	1.8	17
1	Modelling of void formation in the subcooled boiling regime in the athlet code to simulate flow instability for research reactors. <i>Nuclear Engineering and Design</i> , 1996 , 167, 175-191	1.8	27