Yuriy Fedorov

List of Publications by Year in descending order

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		840776	996975
57	368	11	15
papers	citations	h-index	g-index
57	57	57	157
all docs	docs citations	times ranked	citing authors

#	Article	IF	CITATIONS
1	Molten-Salt Reactor for Nuclear Fuel Cycle Closure on All Actinides. Atomic Energy, 2019, 125, 279-283.	0.4	20
2	Simulation of the dynamic beyond-design-basis regime in an extraction–scrubbing unit consisting of mixer–settlers or centrifugal contactors in reprocessing spent nuclear fuel from nuclear power plants and of impurex process. Radiochemistry, 2017, 59, 469-481.	0.7	1
3	Influence of temperature on Pu(IV) extraction from nitric acid solutions into 30% TBP, considered within the framework of A.M. Rozen's model and of a new (multireaction) model. Radiochemistry, 2017, 59, 351-353.	0.7	1
4	Fast-spectrum, liquid-fueled reactors. , 2017, , 375-433.		1
5	Uranium-Plutonium Fuel Cycle of a Fast Molten-Salt Reactor. Atomic Energy, 2016, 121, 63-69.	0.4	6
6	Dissolution of WWER-1000 spent nuclear fuel in a weakly acidic solution of iron nitrate and recovery of actinides and rare earth elements with TBP solutions. Radiochemistry, 2016, 58, 265-270.	0.7	8
7	A New Approach to Simulation of Extraction Equilibria in the PUREX Process. Solvent Extraction and Ion Exchange, 2015, 33, 362-384.	2.0	17
8	Development and trials of a process for extraction recovery of 99Mo for medical purposes from dissolved irradiated uranium targets. Radiochemistry, 2015, 57, 292-306.	0.7	3
9	Interaction of zirconium with pertechnetic acid in the course of extraction from nitric acid solutions with tributyl phosphate in diluent in the presence of uranyl nitrate at various temperatures and its mathematical description. Radiochemistry, 2015, 57, 273-284.	0.7	11
10	Salt-Free Reprocessing of Spent Carbide Fuel for the Example of AM-Reactor Fuel. Atomic Energy, 2015, 117, 394-408.	0.4	0
11	Thermochemical embrittlement of the zirconium cladding of a fuel rod and oxidative recrystallization of the fuel material in the course of spent nuclear fuel reprocessing. Radiochemistry, 2015, 57, 98-102.	0.7	8
12	Influence of temperature on the extraction of actinides from nitric acid solutions into 30% TBP: Description within the framework of a new model. Radiochemistry, 2015, 57, 136-142.	0.7	10
13	Preliminary studies of dynamic regimes in the extraction-scrubbing groups of units of the experimental and demonstration center (EDC) using mathematical modeling. Radiochemistry, 2014, 56, 583-592.	0.7	7
14	Ways of technetium and neptunium localization in extraction reprocessing of spent nuclear fuel from nuclear power plants. Radiochemistry, 2014, 56, 501-514.	0.7	14
15	Mathematical description of the extraction of impurity acids with tributyl phosphate in diluent and of the coextraction of the acid anions with uranyl nitrate. Radiochemistry, 2014, 56, 167-172.	0.7	8
16	A new approach to mathematical description of the extraction of HNO3, U(VI), and other hexavalent actinides with tributyl phosphate in a mixture with paraffins. Radiochemistry, 2013, 55, 291-297.	0.7	11
17	New data on joint extraction of actinide nitrates and some acids with tributyl phosphate diluted with paraffins and mathematical description of the process by A.M. Rozen's Model. Radiochemistry, 2013, 55, 190-202.	0.7	12
18	Extraction of tetravalent actinides and zirconium from nitric acid solutions with tributyl phosphate in diluent in the presence of uranyl nitrate and its mathematical description within the framework of the new approach. Radiochemistry, 2013, 55, 481-485.	0.7	12

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19	Batching of spent AMB nuclear fuel for reprocessing at the industrial association mayak. Atomic Energy, 2013, 114, 344-354.	0.4	5
20	Possibility of using a mixture of enriched regenerated uranium and regenerated plutonium for 100% VVER-1000 core fuel load. Atomic Energy, 2013, 113, 383-391.	0.4	8
21	Extraction of nitric acid and uranyl nitrate with tributyl phosphate in diluent in the presence of salting-out agents and its mathematical description. Radiochemistry, 2013, 55, 369-376.	0.7	7
22	Extraction of molybdenum from nitric acid solutions with solutions of dibutyl hydrogen phosphate zirconium salt. Radiochemistry, 2012, 54, 25-33.	0.7	1
23	A spectroscopic study of molybdenum extracts in organic solutions of dibutyl hydrogen phosphate in equilibrium with aqueous nitric acid solutions. Radiochemistry, 2011, 53, 619-632.	0.7	3
24	Problems of modernization of spent nuclear fuel extraction processing. Russian Journal of General Chemistry, 2011, 81, 1932-1948.	0.8	9
25	Synergistic extraction of REE and TPE from HNO3 solutions with a mixture of CCD and HDBP ZS (1: 6) in a polar diluent. Radiochemistry, 2010, 52, 60-64.	0.7	1
26	Extraction of molybdenum from supersaturated solutions in nitric acid with tributyl phosphate solutions. Radiochemistry, 2010, 52, 180-188.	0.7	5
27	Extraction of Mo from solutions in HNO3 and other mineral acids with solutions of dibutyl hydrogen phosphate in a diluent. Radiochemistry, 2010, 52, 408-417.	0.7	4
28	Development, analysis, and simulation of a technological structure for reprocessing irradiated nuclear fuel from nuclear power plants by water-extraction methods. Atomic Energy, 2009, 107, 333-347.	0.4	13
29	Dibutyl phosphoric acid and its acid zirconium salt as an extractant for the separation of transplutonium elements and rare earths and for their partitioning. Journal of Radioanalytical and Nuclear Chemistry, 2009, 279, 193-208.	1.5	7
30	Synergistic extraction of REE and TPE from HNO3 solutions with a mixture of CCD and HDPB ZS $(1:8)$ in a polar diluent. Radiochemistry, 2009, 51, 30-33.	0.7	2
31	Interaction of HDBP with Zr in extraction. Radiochemistry, 2009, 51, 149-155.	0.7	2
32	Treatment of gas-air flows to remove radioiodine using metallic copper. Radiochemistry, 2009, 51, 409-411.	0.7	6
33	Neutron-physical characteristics of a VVER-1000 core with 100% fuel load consisting of a mixture of recovered uranium and plutonium and enriched uranium. Atomic Energy, 2008, 104, 257-261.	0.4	14
34	Extraction of zirconium with tributyl phosphate from nitric acid solutions. Radiochemistry, 2008, 50, 256-260.	0.7	8
35	Synergistic extraction of REE and TPE from HNO3 solutions with a CCD-HDBP ZS $(1:12)$ mixture in a polar diluent. Radiochemistry, 2008, 50, 621-625.	0.7	1
36	Extraction of alkaline-earth elements from nitric acid with a solution of HDBP zirconium salt. Radiochemistry, 2007, 49, 251-255.	0.7	1

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37	Combined processing scheme of WWER-1000 spent nuclear fuel: 1. Thermochemical breaking-up of fuel claddings and voloxidation of fuel. Radiochemistry, 2007, 49, 380-385.	0.7	8
38	Synergic extraction of Ce(III) from HNO3 solutions with a mixture of CCD and HDBP ZS in a polar diluent. Radiochemistry, 2007, 49, 386-390.	0.7	2
39	Synergistic extraction of REE and TPE from nitric acid solutions with chlorinated cobalt dicarbollide-dibutyl hydrogen phosphate mixture in a polar diluent. Radiochemistry, 2007, 49, 607-612.	0.7	1
40	Combined processing scheme of WWER-1000 spent nuclear fuel: 2. Experimental trial of extraction processing of fluoride cinder. Radiochemistry, 2007, 49, 613-617.	0.7	0
41	Extraction of HNO3 with solutions of zirconium salt of dibutyl hydrogen phosphate in 30% tributyl phosphate and in xylene. Radiochemistry, 2006, 48, 267-271.	0.7	2
42	The influence of dibutyl phosphoric acid on zirconium extraction with diluted tributyl phosphate and the role of uranyl nitrate. European Physical Journal D, 2006, 56, D501-D508.	0.4	0
43	The influence of tributyl phosphate on molybdenum extraction with solutions of dibutyl phosphoric acid. European Physical Journal D, 2006, 56, D509-D517.	0.4	2
44	The influence of tributyl phosphate on molybdenum extraction with solutions of dibutyl phosphoric acid. European Physical Journal D, 2006, 56, D509-D517.	0.4	5
45	Neutron-physical characteristics of a VVÉR core with 100% load of reprocessed uranium and plutonium fuel. Atomic Energy, 2006, 101, 863-868.	0.4	19
46	Use of Recovered Uranium and Plutonium in Thermal Reactors. Atomic Energy, 2005, 99, 572-576.	0.4	16
47	Mathematical Simulation of Steady-State Concentration Profiles in Extraction Cascades for Reprocessing NPP Spent Fuel Using TBP-Compatible Processes (SUPERPUREX). Radiochemistry, 2004, 46, 149-156.	0.7	11
48	Partitioning of High-Level Waste with an Extractant Based on Chlorinated Cobalt Dicarbollide and Dibutylphosphoric Acid Zirconium Salt. Radiochemistry, 2003, 45, 577-580.	0.7	2
49	Methods for Recovering Molybdenum from High-Level Raffinate of Extraction Reprocessing of NPP Spent Fuel. Radiochemistry, 2003, 45, 581-590.	0.7	5
50	Effect of TBP on Extraction of REE and TPE with 4d Element Salts of Dibutylphosphoric Acid. Radiochemistry, 2003, 45, 591-595.	0.7	0
51	Effect of TBP on Extraction of REE and TPE from Nitric Acid Solutions with Dibutylphosphoric Acid and Its Zirconium Salt. Radiochemistry, 2003, 45, 596-601.	0.7	4
52	Extraction of transplutionium and rare-earth elements, molybdenum and iron with zirconium salt of dibutyl phosphoric acid. European Physical Journal D, 2003, 53, A479-A486.	0.4	4
53	TPE/REE separation with the use of zirconium salt of HDBP. European Physical Journal D, 2003, 53, A595-A601.	0.4	8
54	Title is missing!. Radiochemistry, 2001, 43, 569-574.	0.7	3

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55	Title is missing!. Radiochemistry, 2001, 43, 562-568.	0.7	2
56	URANIUM(VT) EXTRACTION BY TBP IN THE PRESENCE OF HDBP. Solvent Extraction and Ion Exchange, 1999, 17, 243-257.	2.0	21
57	Interaction of nitric acid and tetravalent actinides with uranium in TBP-phase. Journal of Radioanalytical and Nuclear Chemistry, 1990, 143, 373-379.	1.5	6