

Tiankai Yao

List of Publications by Year in descending order

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75
papers

2,124
citations

279487

23
h-index

253896

43
g-index

81
all docs

81
docs citations

81
times ranked

2856
citing authors

#	ARTICLE	IF	CITATIONS
1	Understanding fission gas bubble distribution, lanthanide transportation, and thermal conductivity degradation in neutron-irradiated U using machine learning. <i>Materials Characterization</i> , 2022, 184, 111657.	1.9	12
2	Grain subdivision and structural modifications by high-energy heavy ions in UO_2 with different initial grain size. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2022, 515, 48-60.	0.6	4
3	Dislocation loop evolution in Kr -irradiated ThO_2 . <i>Journal of the American Ceramic Society</i> , 2022, 105, 5419-5435.	1.9	11
4	Small-scale mechanical testing and characterization of fuel cladding chemical interaction between HT9 cladding and advanced U-based metallic fuel alloy. <i>Journal of Nuclear Materials</i> , 2022, 566, 153754.	1.3	4
5	Transmission Electron Microscopy based Characterization of a U-20Pu-10Zr Fuel Irradiated in Experimental Breeder Reactor-II. <i>Journal of Nuclear Materials</i> , 2022, 568, 153846.	1.3	2
6	Enhanced crevice corrosion of stainless steel 316 by degradation of Cr-containing hollandite crevice former. <i>Corrosion Science</i> , 2022, 205, 110462.	3.0	2
7	Aluminum-doped U_3Si_2 composite fuels with enhanced oxidation resistance. <i>Journal of Alloys and Compounds</i> , 2021, 853, 157319.	2.8	9
8	Out-of-pile and postirradiated examination of lanthanide and lanthanide-palladium interactions for metallic fuel. <i>Journal of Nuclear Materials</i> , 2021, 544, 152727.	1.3	15
9	Fuel-cladding chemical interaction of a prototype annular U-10Zr fuel with Fe-12Cr ferritic/martensitic HT-9 cladding. <i>Journal of Nuclear Materials</i> , 2021, 544, 152588.	1.3	17
10	3Y-TZP Toughened and Oxidation-resistant U_3Si_2 Composites for Accident Tolerant Fuels. <i>Journal of Nuclear Materials</i> , 2021, 544, 152691.	1.3	7
11	Magnetic, transport and thermal properties of γ -phase UZr_2 . <i>Philosophical Magazine Letters</i> , 2021, 101, 1-11.	0.5	5
12	Fabrication and thermophysical properties of UO_2 - UB_2 and UO_2 - UB_4 composites sintered via spark plasma sintering. <i>Journal of Nuclear Materials</i> , 2021, 544, 152690.	1.3	12
13	Indirect characterization of point defects in proton irradiated ceria. <i>Materialia</i> , 2021, 15, 101019.	1.3	16
14	Understanding spinodal and binodal phase transformations in U-50Zr. <i>Materialia</i> , 2021, 16, 101092.	1.3	14
15	A systematic study of lanthanide titanates ($\text{A}_2\text{Ti}_2\text{O}_7$) chemical durability: corrosion mechanisms and control parameters. <i>Corrosion Science</i> , 2021, 185, 109394.	3.0	13
16	TEM characterization of dislocation loops in proton irradiated single crystal ThO_2 . <i>Journal of Nuclear Materials</i> , 2021, 552, 152998.	1.3	16
17	In situ monitoring of microstructure evolution during thermal processing of uranium-zirconium alloys using laser-generated ultrasound. <i>Journal of Nuclear Materials</i> , 2021, 553, 153005.	1.3	9
18	Spark plasma sintering-densified vanadinite apatite-based chlorine waste forms with high thermal stability and chlorine confinement. <i>Journal of Nuclear Materials</i> , 2020, 528, 151857.	1.3	10

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19	Lattice strain mapping of cracks and indentations in UO ₂ using synchrotron microdiffraction. <i>Journal of Nuclear Materials</i> , 2020, 529, 151943.	1.3	4
20	Thermophysical and mechanical property assessment of UB ₂ and UB ₄ sintered via spark plasma sintering. <i>Journal of Alloys and Compounds</i> , 2020, 818, 153216.	2.8	29
21	Reply to: How much does corrosion of nuclear waste matrices matter. <i>Nature Materials</i> , 2020, 19, 962-963.	13.3	7
22	UO ₂ + 5 vol% ZrB ₂ nano composite nuclear fuels with full boron retention and enhanced oxidation resistance. <i>Ceramics International</i> , 2020, 46, 26486-26491.	2.3	2
23	Corrosion interactions between stainless steel and lead vanado-iodoapatite nuclear waste form part I. <i>Npj Materials Degradation</i> , 2020, 4, .	2.6	8
24	Corrosion interactions between stainless steel and lead vanado-iodoapatite nuclear waste form part II. <i>Npj Materials Degradation</i> , 2020, 4, .	2.6	7
25	Nanoscale TiO ₂ coating improves water stability of Cs ₂ SnCl ₆ . <i>MRS Communications</i> , 2020, 10, 687-694.	0.8	1
26	The influence of lattice defects, recombination, and clustering on thermal transport in single crystal thorium dioxide. <i>APL Materials</i> , 2020, 8, .	2.2	32
27	Degradation mechanism of lead-vanado-iodoapatite in NaCl solution. <i>Corrosion Science</i> , 2020, 172, 108720.	3.0	3
28	¹³⁵ U and ¹³⁷ U-Zr ₂ in neutron irradiated U-10Zr annular metallic fuel. <i>Journal of Nuclear Materials</i> , 2020, 542, 152536.	1.3	28
29	Combining mesoscale thermal transport and x-ray diffraction measurements to characterize early-stage evolution of irradiation-induced defects in ceramics. <i>Acta Materialia</i> , 2020, 193, 61-70.	3.8	25
30	Development of a grain growth model for U ₃ Si ₂ using experimental data, phase field simulation and molecular dynamics. <i>Journal of Nuclear Materials</i> , 2020, 532, 152069.	1.3	13
31	On spinodal-like phase decomposition in U-50Zr alloy. <i>Materialia</i> , 2020, 9, 100592.	1.3	20
32	Self-accelerated corrosion of nuclear waste forms at material interfaces. <i>Nature Materials</i> , 2020, 19, 310-316.	13.3	61
33	Spark plasma sintering (SPS) densified U ₃ Si ₂ pellets: Microstructure control and enhanced mechanical and oxidation properties. <i>Journal of Alloys and Compounds</i> , 2020, 825, 154022.	2.8	40
34	U ₃ Si ₂ and UO ₂ composites densified by spark plasma sintering for accident-tolerant fuels. <i>Journal of Nuclear Materials</i> , 2020, 534, 152147.	1.3	29
35	The grain-size effect on thermal conductivity of uranium dioxide. <i>Journal of Applied Physics</i> , 2019, 126, .	1.1	20
36	Nano- and micro-indentation testing of sintered UO ₂ fuel pellets with controlled microstructure and stoichiometry. <i>Journal of Nuclear Materials</i> , 2019, 516, 169-177.	1.3	30

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37	In situ Investigation of Water Interaction with Lead-Free All Inorganic Perovskite (Cs_2SnCl_6). Journal of Physical Chemistry C, 2019, 123, 9575-9581.	1.5	23
38	Chemical Durability and Dissolution Kinetics of Iodoapatite in Aqueous Solutions. ACS Earth and Space Chemistry, 2019, 3, 452-462.	1.2	16
39	Deciphering the degradation mechanism of the lead-free all inorganic perovskite Cs_2SnI_6 . Npj Materials Degradation, 2019, 3, .	2.6	25
40	Oxygen point defect accumulation in single-phase U_2O_7 . Physical Review Materials, 2019, 3, .	0.9	10
41	Thermally-Conductive and Mechanically-Robust Graphene Nanoplatelet Reinforced UO_2 Composite Nuclear Fuels. Scientific Reports, 2018, 8, 2987.	1.6	19
42	Mechanism of iodine release from iodoapatite in aqueous solution. RSC Advances, 2018, 8, 3951-3957.	1.7	22
43	Radiation-induced amorphization of Langasite $\text{La}_3\text{Ga}_5\text{SiO}_{14}$. Journal of Nuclear Materials, 2018, 500, 50-55.	1.3	7
44	Tunable optical properties and stability of lead free all inorganic perovskites (Cs_2SnCl_6). Journal of Materials Chemistry A, 2018, 6, 2577-2584.	5.2	55
45	Dense nanocrystalline UO_2 fuel pellets synthesized by high pressure spark plasma sintering. Journal of the American Ceramic Society, 2018, 101, 1105-1115.	1.9	21
46	Radiation-induced grain subdivision and bubble formation in U_3Si_2 at LWR temperature. Journal of Nuclear Materials, 2018, 498, 169-175.	1.3	25
47	The thermal stability and consolidation of perovskite variant Cs_2SnCl_6 using spark plasma sintering. Journal of the American Ceramic Society, 2018, 101, 2060-2065.	1.9	15
48	In-situ TEM study of radiation-induced amorphization and recrystallization of hydroxyapatite. Journal of Nuclear Materials, 2018, 512, 307-313.	1.3	15
49	In-situ TEM study of the ion irradiation behavior of U_3Si_2 and U_3Si_5 . Journal of Nuclear Materials, 2018, 511, 56-63.	1.3	12
50	Nano-crystallization induced by high-energy heavy ion irradiation in UO_2 . Scripta Materialia, 2018, 155, 169-174.	2.6	25
51	Consolidation of commercial-size UO_2 fuel pellets using spark plasma sintering and microstructure/microchemical analysis. MRS Communications, 2018, 8, 979-987.	0.8	8
52	In situ synchrotron investigation of grain growth behavior of nano-grained UO_2 . Scripta Materialia, 2017, 131, 29-32.	2.6	16
53	Influence of grain growth on the structural properties of the nanocrystalline $\text{Gd}_2\text{Ti}_2\text{O}_7$. Journal of Nuclear Materials, 2017, 487, 373-379.	1.3	27
54	Microstructure control of macroscopic graphene paper by electrospray deposition and its effect on thermal and electrical conductivities. Applied Physics Letters, 2017, 110, .	1.5	12

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55	Fabrication of lanthanum-doped thorium dioxide by high-energy ball milling and spark plasma sintering. <i>Journal of Nuclear Materials</i> , 2017, 485, 207-215.	1.3	11
56	Grain growth and pore coarsening in dense nano-crystalline UO_2 fuel pellets. <i>Journal of the American Ceramic Society</i> , 2017, 100, 2651-2658.	1.9	26
57	Bubble morphology in U_3Si_2 implanted by high-energy Xe ions at 300°C. <i>Journal of Nuclear Materials</i> , 2017, 495, 146-153.	1.3	33
58	Correlation between crystallographic orientation and surface faceting in UO_2 . <i>Journal of Nuclear Materials</i> , 2016, 478, 176-184.	1.3	10
59	Radiation-induced amorphization of Ce-doped $Mg_2Y_8(SiO_4)_6O_2$ silicate apatite. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2016, 379, 102-106.	0.6	12
60	TiO_2 doped UO_2 fuels sintered by spark plasma sintering. <i>Journal of Nuclear Materials</i> , 2016, 469, 251-261.	1.3	31
61	Radiation Stability of Spark Plasma Sintered Lead Vanadate Iodoapatite. <i>Journal of the American Ceramic Society</i> , 2015, 98, 3361-3366.	1.9	15
62	Dense Iodoapatite Ceramics Consolidated by Low Temperature Spark Plasma Sintering. <i>Journal of the American Ceramic Society</i> , 2015, 98, 3733-3739.	1.9	17
63	Graphene-based sorbents for iodine-129 capture and sequestration. <i>Carbon</i> , 2015, 90, 1-8.	5.4	91
64	Highly thermally conductive and mechanically strong graphene fibers. <i>Science</i> , 2015, 349, 1083-1087.	6.0	564
65	Metallization and softening of B_6O at high pressure. <i>Journal of Alloys and Compounds</i> , 2014, 600, 71-77.	2.8	6
66	Facile low temperature solid state synthesis of iodoapatite by high-energy ball milling. <i>RSC Advances</i> , 2014, 4, 38718-38725.	1.7	19
67	Advanced Phase Change Composite by Thermally Annealed Defect-Free Graphene for Thermal Energy Storage. <i>ACS Applied Materials & Interfaces</i> , 2014, 6, 15262-15271.	4.0	113
68	Flexible, thorn-like ZnO-multiwalled carbon nanotube hybrid paper for efficient ultraviolet sensing and photocatalyst applications. <i>Nanoscale</i> , 2014, 6, 13630-13636.	2.8	44
69	Bulk Iodoapatite Ceramic Densified by Spark Plasma Sintering with Exceptional Thermal Stability. <i>Journal of the American Ceramic Society</i> , 2014, 97, 2409-2412.	1.9	43
70	First-principles study of structural stability, elastic and electronic properties of ternary rare earth-transition metal Borides and carbides (RT_xZ , R=Sc, Y, and La, T=Pt and Pd, Z=B and C, and x=2, 3). <i>Tj ETQq</i> 0.0 0 rgBTi/Overlock	0.0	0
71	Structural and relative stabilities, electronic properties and possible reactive routing of osmium and ruthenium borides from first-principles calculations. <i>Dalton Transactions</i> , 2013, 42, 7041.	1.6	31
72	Understanding the mechanical properties of vanadium carbides: Nano-indentation measurement and first-principles calculations. <i>Journal of Alloys and Compounds</i> , 2013, 548, 60-64.	2.8	90

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73	Does the real ReN_2 have the MoS_2 structure?. Physical Chemistry Chemical Physics, 2013, 15, 183-187.	1.3	22
74	Structural stability, phase transition, and mechanical and electronic properties of transition metal nitrides MN (M=Tc, Re, Os, and Ir): First-principles calculations. Computational Materials Science, 2012, 56, 116-121.	1.4	20
75	A universal trend of structural, mechanical and electronic properties in transition metal (M=V, Nb, Tj ETQq1 1 0.784314 rgBT /Overlo	1.4	33