

Rafael Juarez

List of Publications by Year in descending order

Source: <https://exaly.com/author-pdf/1268389/publications.pdf>

Version: 2024-02-01

42
papers

431
citations

933447

10
h-index

839539

18
g-index

42
all docs

42
docs citations

42
times ranked

254
citing authors

#	ARTICLE	IF	CITATIONS
1	Shielding conceptual designs of ITER TCP ports to protect electronics. Fusion Engineering and Design, 2022, 176, 113016.	1.9	2
2	Development of radiation sources for nuclear analysis beyond ITER bio-shield: SRC-UNED code. Computer Physics Communications, 2022, 275, 108309.	7.5	8
3	Improved radiation shielding analysis considering vector calculus. International Journal of Energy Research, 2021, 45, 11904-11915.	4.5	0
4	A full and heterogeneous model of the ITER tokamak for comprehensive nuclear analyses. Nature Energy, 2021, 6, 150-157.	39.5	34
5	Nuclear data for D1SUNED for the study of ITER planned in-situ maintenance dose scenarios. Fusion Engineering and Design, 2021, 170, 112646.	1.9	6
6	Integral modelling of the ITER cooling water systems radiation source for applications outside of the Bio-shield. Fusion Engineering and Design, 2021, 171, 112575.	1.9	10
7	D1SUNED system for the determination of decay photon related quantities. Fusion Engineering and Design, 2020, 151, 111399.	1.9	42
8	Nuclear analysis of the ITER torus cryopumps. Nuclear Fusion, 2019, 59, 106045.	3.5	8
9	Shielding proposal to mitigate the PFC #2 heating due to 16N decay photon radiation from the activated cooling water. Fusion Engineering and Design, 2019, 148, 111298.	1.9	3
10	Standardized integration of ITER diagnostics Equatorial Port Plugs. Fusion Engineering and Design, 2019, 146, 1548-1552.	1.9	14
11	Assessment and optimization of MCNP memory management for detailed geometry of nuclear fusion facilities. Fusion Engineering and Design, 2018, 136, 386-389.	1.9	5
12	The ITER tokamak neutronics reference model C-Model. Fusion Engineering and Design, 2018, 136, 742-746.	1.9	52
13	MCNP model of the ITER Tokamak Complex. Fusion Engineering and Design, 2018, 136, 859-863.	1.9	16
14	The use of the long modular diagnostics shield module to mitigate shutdown dose rates in the ITER diagnostics equatorial ports. Nuclear Fusion, 2018, 58, 056015.	3.5	10
15	ITER plasma source and building modelling to produce radiation maps. Nuclear Fusion, 2018, 58, 126012.	3.5	15
16	Shutdown dose rate mitigation in the ITER upper ports. Fusion Engineering and Design, 2018, 136, 228-232.	1.9	10
17	Scoping studies of shielding to reduce the shutdown dose rates in the ITER ports. Nuclear Fusion, 2018, 58, 076018.	3.5	7
18	Neutronic effects of diagnostic shield module length on radiation environment of ITER diagnostic generic upper port plug. Fusion Engineering and Design, 2018, 136, 920-924.	1.9	3

#	ARTICLE	IF	CITATIONS
19	Update in the nuclear responses of the European TBMs for ITER during operation and shutdown. Fusion Engineering and Design, 2018, 134, 92-96.	1.9	6
20	Functional materials for ITER diagnostic systems – Radiation aspects. Fusion Engineering and Design, 2017, 125, 277-282.	1.9	4
21	Radiation In-Port Cross-Talks for ITER Port Diagnostics. Fusion Science and Technology, 2017, 72, 559-565.	1.1	3
22	CAD-Based Shielding Analysis for ITER Port Diagnostics. EPJ Web of Conferences, 2017, 153, 03010.	0.3	1
23	Shutdown dose rate analysis of European test blanket modules shields in ITER Equatorial Port #16. Fusion Engineering and Design, 2016, 109-111, 1554-1558.	1.9	12
24	Status of the ITER tokamak nuclear shielding and radiological protection design. Fusion Engineering and Design, 2016, 109-111, 666-672.	1.9	8
25	Study of shielding options for lower ports for mitigation of neutron environment and shutdown dose inside the ITER cryostat. Fusion Engineering and Design, 2016, 109-111, 1408-1411.	1.9	8
26	Development of the R2SUNED Code System for Shutdown Dose Rate Calculations. IEEE Transactions on Nuclear Science, 2016, 63, 375-384.	2.0	39
27	Shielding proposal to reduce cross-talk from ITER lower port to equatorial port. Fusion Engineering and Design, 2015, 101, 67-72.	1.9	5
28	Nuclear analysis of the ITER Cryopump Ports. Fusion Engineering and Design, 2015, 98-99, 1561-1565.	1.9	6
29	Global shutdown dose rate maps for a DEMO conceptual design. Fusion Engineering and Design, 2015, 98-99, 1524-1527.	1.9	9
30	Thermo-fluid dynamics and corrosion analysis of a self cooled lead lithium blanket for the HiPER reactor. Nuclear Fusion, 2015, 55, 093003.	3.5	3
31	Shutdown dose rates at ITER equatorial ports considering radiation cross-talk from torus cryopump lower port. Fusion Engineering and Design, 2015, 100, 501-506.	1.9	20
32	Neutronics and activation of the preliminary reaction chamber of HiPER reactor based in a SCLL blanket. Fusion Engineering and Design, 2013, 88, 2373-2377.	1.9	4
33	Design and analysis of helium Brayton power cycles for HiPER reactor. Fusion Engineering and Design, 2013, 88, 2679-2683.	1.9	13
34	Silica final lens performance in laser fusion facilities: HiPER and LIFE. Nuclear Fusion, 2013, 53, 013010.	3.5	10
35	Studies of a self-cooled lead lithium blanket for HiPER reactor. EPJ Web of Conferences, 2013, 59, 11005.	0.3	1
36	Radioprotection Analysis for the High Energy Beam Transport Line of the Accelerator Facility of IFMIF. Fusion Science and Technology, 2012, 62, 240-245.	1.1	5

#	ARTICLE	IF	CITATIONS
37	Advances in neutronics and radiological protection of HiPER 4a. Fusion Engineering and Design, 2012, 87, 336-343.	1.9	2
38	Dose rates evaluation of HiPER facility. Fusion Engineering and Design, 2011, 86, 694-698.	1.9	5
39	Waste Management Assessment of Candidate Materials for HiPER Reaction Chamber. Fusion Science and Technology, 2011, 60, 579-584.	1.1	3
40	Lifetime of silica final lenses subject to HiPER irradiation conditions. Proceedings of SPIE, 2011, , .	0.8	7
41	HiPER laser reference design. , 2011, , .		7
42	IFE plant technology overview and contribution to HiPER proposal. , 2011, , .		5