Akira Hasegawa

List of Publications by Year
in descending order

218 papers	7,403 citations	61984 43 h-index	62596 80 g-index
220 all docs	220 docs citations	220 times ranked	2806 citing authors

#	Article	IF	Citations
1	Neutron irradiation-enhanced grain growth in tungsten and tungsten alloys. Journal of Alloys and Compounds, 2022, 901, 163419.	5.5	21
2	Effects of microstructural anisotropy and helium implantation on tensile properties of powder-metallurgy processed tungsten plates. Nuclear Materials and Energy, 2022, 30, 101122.	1.3	0
3	Tungstenâ€“tantalum alloys for fusion reactor applications. Journal of Nuclear Materials, 2022, 566, 153740.	2.7	15
4	Irradiation damages of structural materials under different irradiation environments. Journal of Nuclear Materials, 2021, 543, 152503.	2.7	11
5	Mechanical properties of tungsten: Recent research on modified tungsten materials in Japan. Journal of Nuclear Materials, 2021, 543, 152506.	2.7	55
6	Thermal diffusivity of irradiated tungsten and tungsten-rhenium alloys. Journal of Nuclear Materials, 2021, 543, 152594.	2.7	13
7	Neutron irradiation tolerance of potassium-doped and rhenium-alloyed tungsten. Journal of Nuclear Materials, 2021, 553, 153009.	2.7	21
8	Helium Effects on Tensile Properties of Powder Metallurgical-Processed Tungsten for Fusion Reactor Applications. Nuclear Materials and Energy, 2021, 29, 101076.	1.3	1
9	Tungsten modified by potassium doping and rhenium addition for fusion reactor applications. Fusion Engineering and Design, 2020, 152, 111445.	1.9	33
10	Neutron irradiation effects on the mechanical properties of powder metallurgical processed tungsten alloys. Journal of Nuclear Materials, 2020, 529, 151910.	2.7	23
11	Fatigue properties of ferritic/martensitic steel after neutron irradiation and helium implantation. Nuclear Materials and Energy, 2020, 24, 100764.	1.3	4
12	Laminated composites using potassium doped tungsten. Fusion Engineering and Design, 2020, 161, 111894.	1.9	1
13	Tensile properties of powder-metallurgical-processed tungsten alloys after neutron irradiation near recrystallization temperatures. Journal of Nuclear Materials, 2020, 542, 152505.	2.7	16
14	Tungsten as a Plasma-Facing Material. , 2020, , 19-53.		10
15	Helium effects on recovery and recrystallization of powder metallurgically processed tungsten. Physica Scripta, 2020, T171, 014016.	2.5	8
16	Effects of Helium and Displacement Damage on Microstructural Evolution in Helium-Implanted Martensitic Steel HCM12A Examined by TEM and Positron Annihilation Lifetime Measurement. , 2020, , .		0
17	Tensile and impact properties of tungsten-rhenium alloy for plasma-facing components in fusion reactor. Fusion Engineering and Design, 2019, 148, 111323.	1.9	27
18	Improvement of impact properties of tungsten by potassium doping. Fusion Engineering and Design, 2019, 140, 48-61.	1.9	24

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19	PHENIX U.S.-Japan Collaboration Investigation of Thermal and Mechanical Properties of Thermal Neutronâ€“Shielded Irradiated Tungsten. Fusion Science and Technology, 2019, 75, 499-509.	1.1	28
20	Behavior of tungsten under irradiation and plasma interaction. Journal of Nuclear Materials, 2019, 519, 334-368.	2.7	129
21	Effects of helium on mechanical properties of tungsten for fusion applications. Nuclear Materials and Energy, 2018, 15, 154-157.	1.3	22
22	Recrystallization behavior of hot-rolled pure tungsten and its alloy plates during high-temperature annealing. Nuclear Materials and Energy, 2018, 15, 158-163.	1.3	71
23	First-principles study of solvent-solute mixed dumbbells in body-centered-cubic tungsten crystals. Journal of Nuclear Materials, 2018, 505, 15-21.	2.7	18
24	Solid state diffusion bonding of doped tungsten alloys with different thermo-mechanical properties. Fusion Engineering and Design, 2018, 136, 76-81.	1.9	8
25	Effect of neutron irradiation on rhenium cluster formation in tungsten and tungsten-rhenium alloys. Journal of Nuclear Materials, 2018, 507, 78-86.	2.7	47
26	Thermal properties of pure tungsten and its alloys for fusion applications. Fusion Engineering and Design, 2018, 132, 1-6.	1.9	64
27	Japanese activities of the R&D on silicon carbide composites in the broader approach period and beyond. Journal of Nuclear Materials, 2018, 511, 582-590.	2.7	6
28	A review of impact properties of tungsten materials. Fusion Engineering and Design, 2018, 135, 196-203.	1.9	30
29	Fatigue life prediction of ferritic/martensitic steels based on universal slope equations and tensile properties obtained using small specimen. Fusion Engineering and Design, 2017, 125, 330-336.	1.9	3
30	Fatigue Properties of SiC/SiC Composites Under Various Loading Modes. Fusion Science and Technology, 2017, , 1-6.	1.1	3
31	Baseline high heat flux and plasma facing materials for fusion. Nuclear Fusion, 2017, 57, 092006.	3.5	141
32	Degradation of tungsten monoblock divertor under cyclic high heat flux loading. Fusion Engineering and Design, 2017, 120, 49-60.	1.9	28
33	Improved structural strength and lifetime of monoblock divertor targets by using doped tungsten alloys under cyclic high heat flux loading. Physica Scripta, 2017, T170, 014011.	2.5	7
34	Feasibility of Utilizing Tungsten Rod for Fusion Reactor Divertor. Fusion Science and Technology, 2017, 72, 673-679.	1.1	17
35	High Temperature Fatigue Life Evaluation Using Small Specimen. Plasma and Fusion Research, 2017, 12, 1405022-1405022.	0.7	4
36	Analysis of the temperature and thermal stress in pure tungsten monoblock during heat loading and the influences of alloying and dispersion strengthening on these responses. Fusion Engineering and Design, 2016, 107, 44-50.	1.9	15

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37	The effect of voids on the hardening of body-centered cubic Fe. Journal of Nuclear Materials, 2016, 471, 233-238.	2.7	15
38	Effect of self-ion irradiation on hardening and microstructure of tungsten. Nuclear Materials and Energy, 2016, 9, 430-435.	1.3	44
39	Neutron energy spectrum influence on irradiation hardening and microstructural development of tungsten. Journal of Nuclear Materials, 2016, 479, 249-254.	2.7	64
40	Suppression of radiation-induced point defects by rhenium and osmium interstitials in tungsten. Scientific Reports, 2016, 6, 36738.	3.3	28
41	Effect of microstructural anisotropy on the mechanical properties of K-doped tungsten rods for plasma facing components. Fusion Engineering and Design, 2016, 109-111, 1549-1553.	1.9	19
42	Neutron irradiation effects on the microstructural development of tungsten and tungsten alloys. Journal of Nuclear Materials, 2016, 471, 175-183.	2.7	151
43	Strain rate dependence of tensile properties of tungsten alloys for plasma-facing components in fusion reactors. Fusion Engineering and Design, 2016, 109-111, 1674-1677.	1.9	26
44	Tensile and fatigue properties of potassium doped and rhenium containing tungsten rods for fusion reactor applications. Fusion Engineering and Design, 2016, 109-111, 1538-1542.	1.9	17
45	Fatigue Crack Initiation of F82H Reduced Activation Ferritic/Martensitic Steel. Fusion Science and Technology, 2015, 68, 601-606.	1.1	0
46	Effect of Non-Uniform Deformation on Low Cycle Fatigue Properties of Electron Beam Weld Joint of F82H Steel. Fusion Science and Technology, 2015, 68, 607-611.	1.1	1
47	Anisotropy in the Mechanical Properties of Potassium and Rhenium Doped Tungsten Alloy Plates for Fusion Reactor Applications. Fusion Science and Technology, 2015, 68, 690-693.	1.1	35
48	Effect of Grain Structure Anisotropy and Recrystallization on Tensile Properties of Swaged Tungsten Rod. Plasma and Fusion Research, 2015, 10, 1405073-1405073.	0.7	8
49	Effects of temperature and strain rate on the tensile properties of potassium-doped tungsten. Journal of Nuclear Materials, 2015, 461, 357-364.	2.7	33
50	Effect of post-weld heat treatment and neutron irradiation on a dissimilar-metal joint between F82H steel and 316L stainless steel. Fusion Engineering and Design, 2015, 98-99, 1968-1972.	1.9	12
51	Migration of rhenium and osmium interstitials in tungsten. Journal of Nuclear Materials, 2015, 467, 418-423.	2.7	46
52	Overview on recent progress toward small specimen test technique. Fusion Engineering and Design, 2015, 98-99, 2089-2093.	1.9	17
53	Stability and mobility of rhenium and osmium in tungsten: first principles study. Modelling and Simulation in Materials Science and Engineering, 2014, 22, 075006.	2.0	81
54	Effect of He implantation on fracture behavior and microstructural evolution in F82H. Journal of Nuclear Materials, 2014, 455, 690-694.	2.7	9

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55	Tensile properties of K-doped Wâ€“3%Re. Fusion Engineering and Design, 2014, 89, 1033-1036.	1.9	24
56	Continuous SiC fiber, CVI SiC matrix composites for nuclear applications: Properties and irradiation effects. Journal of Nuclear Materials, 2014, 448, 448-476.	2.7	368
57	Effects of tool rotation speed on the mechanical properties and microstructure of friction stir welded ODS steel. Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing, 2014, 595, 291-296.	5.6	38
58	Microstructural development of tungsten and tungstenâ€“rhenium alloys due to neutron irradiation in HFIR. Journal of Nuclear Materials, 2014, 455, 460-463.	2.7	118
59	Neutron irradiation effects on tungsten materials. Fusion Engineering and Design, 2014, 89, 1568-1572.	1.9	167
60	Evaluation of irradiation hardening of proton irradiated stainless steels by nanoindentation. Journal of Nuclear Materials, 2014, 446, 142-147.	2.7	70
61	Microstructure development of dispersion-strengthened tungsten due to neutron irradiation. Journal of Nuclear Materials, 2014, 449, 213-218.	2.7	34
62	Structural Optimization of the Blanket First Wall to Reduce Thermal Stress Using the Taguchi Method. Plasma and Fusion Research, 2014, 9, 1405143-1405143.	0.7	0
63	Effect of helium on fatigue crack growth and life of reduced activation ferritic/martensitic steel. Journal of Nuclear Materials, 2013, 442, S43-S47.	2.7	6
64	Study on electron beam weld joints between pure vanadium and SUS316L stainless steel. Journal of Nuclear Materials, 2013, 442, S562-S566.	2.7	23
65	Effect of heat treatment on bend stress relaxation of pure tungsten. Fusion Engineering and Design, 2013, 88, 1735-1738.	1.9	5
66	Recent progress in R&D on tungsten alloys for divertor structural and plasma facing materials. Journal of Nuclear Materials, 2013, 442, S181-S189.	2.7	272
67	Development of fatigue life evaluation method using small specimen. Journal of Nuclear Materials, 2013, 441, 125-132.	2.7	23
68	Tritium trapping in silicon carbide in contact with solid breeder under high flux isotope reactor irradiation. Journal of Nuclear Materials, 2013, 442, S497-S500.	2.7	4
69	Property change of advanced tungsten alloys due to neutron irradiation. Journal of Nuclear Materials, 2013, 442, S273-S276.	2.7	95
70	Neutron Irradiation Behavior of Tungsten. Materials Transactions, 2013, 54, 466-471.	1.2	96
71	Silicon Carbide and Silicon Carbide Composites for Fusion Reactor Application. Materials Transactions, 2013, 54, 472-476.	1.2	34
72	Bend Stress Relaxation of Pure Tungsten. Plasma and Fusion Research, 2013, 8, 1405006-1405006.	0.7	2

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73	Effects of Re Content and Fabrication Process on Microstructural Changes and Hardening in Neutron Irradiated Tungsten. Materials Transactions, 2012, 53, 2145-2150.	1.2	84
74	Study on Fatigue Life Evaluation Using Small Specimens for Testing Neutron-Irradiated Materials. Journal of Nuclear Science and Technology, 2011, 48, 60-64.	1.3	12
75	Fatigue Crack Initiation in Proton-Irradiated Austenitic Stainless Steel. Journal of Nuclear Science and Technology, 2011, 48, 1265-1271.	1.3	3
76	Indentation Properties of Silicon Carbide after Neutron Irradiation and Helium Implantation. IOP Conference Series: Materials Science and Engineering, 2011, 18, 162007.	0.6	5
77	Study on Compatibility Between Silicon Carbide and Solid Breeding Materials Under Neutron Irradiation. Fusion Science and Technology, 2011, 60, 288-291.	1.1	10
78	Midterm Summary of Japan-US Fusion Cooperation Program TITAN. Fusion Science and Technology, 2011, 60, 321-328.	1.1	7
79	Recent progress of tungsten R&D for fusion application in Japan. Physica Scripta, 2011, T145, 014029.	2.5	39
80	Microstructure Development in Neutron Irradiated Tungsten Alloys. Materials Transactions, 2011, 52, 1447-1451.	1.2	112
81	Effect of PWHT on the Mechanical and Metallographical Properties of a Dissimilar-Metal Weld Joint of F82H and SUS316L Steels. Fusion Science and Technology, 2011, 60, 334-338.	1.1	8
82	Study on Dissimilar-Material Welding with Vanadium and Austenitic Stainless Steel. Fusion Science and Technology, 2011, 60, 417-421.	1.1	3
83	Effect of specimen shape on micro-crack growth behavior under fatigue in reduced activation ferritic/martensitic steel. Journal of Nuclear Materials, 2011, 417, 131-134.	2.7	10
84	Property change mechanism in tungsten under neutron irradiation in various reactors. Journal of Nuclear Materials, 2011, 417, 491-494.	2.7	99
85	Helium effects on the tensile property of 316FR stainless steel at 650 and 750 Â°C. Journal of Nuclear Materials, 2011, 417, 1030-1033.	2.7	3
86	Study on stress relaxation behavior of silicon carbide by BSR method. Journal of Nuclear Materials, 2011, 417, 356-358.	2.7	8
87	Mechanical properties of advanced SiC fiber composites irradiated at very high temperatures. Journal of Nuclear Materials, 2011, 417, 416-420.	2.7	43
88	Evaluation of bonding strength between yttria coating and vanadium alloys for development of self-cooled blanket. Journal of Nuclear Materials, 2011, 417, 1253-1256.	2.7	4
89	High-Temperature Helium Embrittlement of 316FR Steel. Journal of Nuclear Science and Technology, 2011, 48, 130-134.	1.3	16
90	Evaluation of adhesive strength between vanadium alloys and yttrium oxide by laser shock spallation method. Materials Research Society Symposia Proceedings, 2011, 1298, 55.	0.1	0

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91	Study on Fatigue Life Evaluation Using Small Specimens for Testing Neutron-Irradiated Materials. Journal of Nuclear Science and Technology, 2011, 48, 60-64.	1.3	2
92	Fatigue Crack Initiation in Proton-Irradiated Austenitic Stainless Steel. Journal of Nuclear Science and Technology, 2011, 48, 1265-1271.	1.3	0
93	High-Temperature Helium Embrittlement of 316FR Steel. Journal of Nuclear Science and Technology, 2011, 48, 130-134.	1.3	0
94	Effect of Specimen Shape on the Low Cycle Fatigue Life of Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 47-52.	1.3	15
95	Fatigue Life Assessment Based on Crack Growth Behavior in Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 457-461.	1.3	6
96	Fatigue Life Assessment Based on Crack Growth Behavior in Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 457-461.	1.3	1
97	Effect of Specimen Shape on the Low Cycle Fatigue Life of Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 47-52.	1.3	3
98	S0306-4-4 Study on Prediction of Fatigue Crack Initiation Life in Austenitic Stainless Steels. The Proceedings of the JSME Annual Meeting, 2010, 2010.1, 49-50.	0.0	0
99	Effects of helium on ductile-brittle transition behavior of reduced-activation ferritic steels after high-concentration helium implantation at high temperature. Journal of Nuclear Materials, 2009, 386-388, 241-244.	2.7	41
100	Evaluation of interface strength between metal and ceramics to be utilized for development of fusion reactor components. Journal of Nuclear Materials, 2009, 386-388, 689-691.	2.7	7
101	Effects of transmutation elements on the microstructural evolution and electrical resistivity of neutron-irradiated tungsten. Journal of Nuclear Materials, 2009, 386-388, 218-221.	2.7	85
102	Compatibility between SiC and Li ceramics for solid breeding blanket system. Journal of Nuclear Materials, 2009, 386-388, 628-630.	2.7	9
103	Recent advances and issues in development of silicon carbide composites for fusion applications. Journal of Nuclear Materials, 2009, 386-388, 622-627.	2.7	124
104	Improvement of Surface Exfoliation Behavior by Helium-ion Bombardment of a Tungsten Alloy Fabricated by Mechanical Alloying. Journal of Nuclear Science and Technology, 2009, 46, 717-723.	1.3	12
105	Mechanical Property Changes and Irradiation Hardening Due to Dissimilar Metal Welding with Reduced Activation Ferritic/Martensitic Steel and 316L Stainless Steel. Fusion Science and Technology, 2009, 56, 318-322.	1.1	11
106	Development of Low-Activation Reinforced Concrete Design Methodology â€" I: Manufacture of Low-Activation Concrete. Nuclear Technology, 2009, 168, 564-570.	1.2	2
107	Development of Low-Activation Reinforced Concrete Design Methodology â€" II: Concrete Activation Analyses of BWR/PWR. Nuclear Technology, 2009, 168, 571-575.	1.2	0
108	MEASUREMENT OF ADHESION STRENGTH OF SOLID-STATE DIFFUSION BONDING BETWEEN NICKEL AND COPPER BY MEANS OF LASER SHOCK SPALLATION METHOD. , 2009, , .		0

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109	Improvement of Surface Exfoliation Behavior by Helium-ion Bombardment of a Tungsten Alloy Fabricated by Mechanical Alloying. Journal of Nuclear Science and Technology, 2009, 46, 717-723.	1.3	2
110	Role of Oxide Layer on Wall Thinning Caused by Liquid Droplet Impingement. , 2009, , .		0
111	Oxidation behavior of SiC/SiC composites for helium cooled solid breeder blanket. Fusion Engineering and Design, 2008, 83, 1490-1494.	1.9	17
112	Development of advanced blanket performance under irradiation and system integration through JUPITER-II project. Fusion Engineering and Design, 2008, 83, 842-849.	1.9	15
113	Development of ultra-fine grained Wâ€“(0.25â€“0.8)wt%TiC and its superior resistance to neutron and 3MeV He-ion irradiations. Journal of Nuclear Materials, 2008, 377, 34-40.	2.7	178
114	Present status of study on development of materials resistant to radiation and beam impact. Journal of Nuclear Materials, 2008, 377, 21-27.	2.7	14
115	Effects of transmutation elements on the defect structure development of W irradiated by protons and neutrons. Journal of Nuclear Materials, 2008, 377, 348-351.	2.7	30
116	Development of Low-Activation Design Method for Reduction of Radioactive Waste Below Clearance Level. , 2008, , .		5
117	Precipitation of Solid Transmutation Elements in Irradiated Tungsten Alloys. Materials Transactions, 2008, 49, 2259-2264.	1.2	62
118	Distribution of Cobalt between MgO-Saturated FeO<I>_x</I>-MgO-CaO-SiO₂ Slag and Fe-Cu-Co Molten Alloy. Materials Transactions, 2008, 49, 2636-2641.	1.2	5
119	EBSD Analysis of Localized Deformation Behavior of Austenitic Steel Irradiated by Light Ion. Materia Japan, 2008, 47, 623-623.	0.1	0
120	Microstructure Evaluation of Advanced W Alloy with Fine Grain Structure Superior to Helium Irradiation Property. Materia Japan, 2008, 47, 633-633.	0.1	0
121	Effects of Transmutation Elements on Neutron Irradiation Hardening of Tungsten. Materials Transactions, 2007, 48, 2399-2402.	1.2	120
122	Effect of Al and Be ions pre-implantation on formation and growth of helium bubbles in SiC/SiC composites. Nuclear Instruments & Methods in Physics Research B, 2007, 256, 669-674.	1.4	1
123	Current status and critical issues for development of SiC composites for fusion applications. Journal of Nuclear Materials, 2007, 367-370, 659-671.	2.7	310
124	Effect of displacement damage up to 50dpa on microstructural development in SiC/SiC composites. Journal of Nuclear Materials, 2007, 367-370, 698-702.	2.7	6
125	Behavior of Cobalt in Iron- and Steel-making Processes. ISIJ International, 2007, 47, 1818-1822.	1.4	1
126	Microstructural development and irradiation hardening of W and Wâ€“(3â€“26) wt%Re alloys after high-temperature neutron irradiation to 0.15â€‰dpa. Nuclear Fusion, 2006, 46, 877-883.	3.5	64

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127	Overview of design and R&D of test blankets in Japan. Fusion Engineering and Design, 2006, 81, 415-424.	1.9	61
128	Plan and Strategy for ITER Blanket Testing in Japan. Fusion Science and Technology, 2005, 47, 1023-1030.	1.1	4
129	Cavity Formation in a SiC/SiC Composite under Simultaneous Irradiation of Hydrogen, Helium and Silicon Ions. Materials Transactions, 2005, 46, 536-542.	1.2	12
130	Improvement of Corrosion Resistance of Vanadium Alloys in High-Temperature Pressurized Water. Materials Transactions, 2005, 46, 517-521.	1.2	2
131	Effects of Helium on Radiation Damage in Fusion Materials 6.Effects of Transmutation Helium on Neutron Irradiation Behavior of Candidate Structural Materials for Fusion Reactor. Journal of Plasma and Fusion Research, 2005, 81, 36-40.	0.4	0
132	Perspectives of SiC-Based Ceramic Composites and Their Applications to Fusion Reactors 4.Neutron Irradiation Effects in SiC and SiC⁄SiC Composites. Journal of Plasma and Fusion Research, 2004, 80, 24-30.	0.4	4
133	Effects of small amount of additional elements on control of interstitial impurities and mechanical properties of Vâ€“4Crâ€“4Tiâ€“Siâ€“Alâ€“Y alloys. Journal of Nuclear Materials, 2004, 326, 1-8.	2.7	13
134	Microstructural development and radiation hardening of neutron irradiated Moâ€“Re alloys. Journal of Nuclear Materials, 2004, 324, 62-70.	2.7	41
135	Issues and advances in SiCf/SiC composites development for fusion reactors. Journal of Nuclear Materials, 2004, 329-333, 56-65.	2.7	178
136	Mechanical properties and microstructures of high-chromium Vâ€“Crâ€“Ti type alloys. Journal of Nuclear Materials, 2004, 329-333, 457-461.	2.7	29
137	Influence of Cr, Ti concentrations on oxidation and corrosion resistance of Vâ€“Crâ€“Ti type alloys. Journal of Nuclear Materials, 2004, 329-333, 452-456.	2.7	17
138	Study on irradiation induced corrosion behavior in austenitic stainless steel using hydrogen-ion bombardment. Journal of Nuclear Materials, 2004, 329-333, 652-656.	2.7	12
139	Effect of He pre-implantation and neutron irradiation on mechanical properties of SiC/SiC composite. Journal of Nuclear Materials, 2004, 329-333, 577-581.	2.7	11
140	Study of hydrogen effects on microstructural development of SiC base materials under simultaneous irradiation with He- and Si-ion irradiation conditions. Journal of Nuclear Materials, 2004, 329-333, 582-586.	2.7	19
141	Synergistic effects of implanted helium and hydrogen and the effect of irradiation temperature on the microstructure of SiC/SiC composites. Journal of Nuclear Materials, 2004, 335, 508-514.	2.7	38
142	Fabrication and Characterization of W-Re-Os Alloys for Studying Transmutation Effects of W in Fusion Reactors. Materials Transactions, 2004, 45, 2657-2660.	1.2	18
143	R&D of A MW-class solid-target for a spallation neutron source. Journal of Nuclear Materials, 2003, 318, 38-55.	2.7	31
144	Synergistic Effect of Displacement Damage, Helium and Hydrogen of Silicon Carbide Composite. Fusion Science and Technology, 2003, 44, 175-180.	1.1	10

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145	Japanese Evaluated Nuclear Data Library Version 3 Revision-3: JENDL-3.3. Journal of Nuclear Science and Technology, 2002, 39, 1125-1136.	1.3	469
146	JENDL-3.2 Covariance File. Journal of Nuclear Science and Technology, 2002, 39, 40-43.	1.3	18
147	Neutron-Induced Fission of²³³U,²³⁸U,²³²Th,²³⁹Pu,²³⁷Np,^{nat}Pb and²⁰⁹Bi Relative to²³⁵U in the Energy Range 1-200 MeV. Journal of Nuclear Science and Technology, 2002, 39, 230-233.	1.3	94
148	Integral Test of JENDL-3.3 with Shielding Benchmarks. Journal of Nuclear Science and Technology, 2002, 39, 841-846.	1.3	4
149	Actinide Neutron-Induced Fission up to 200 MeV. Journal of Nuclear Science and Technology, 2002, 39, 80-83.	1.3	1
150	Neutron Scattering on238U and232Th. Journal of Nuclear Science and Technology, 2002, 39, 148-151.	1.3	4
151	Promise and challenges of SiCf/SiC composites for fusion energy applications. Journal of Nuclear Materials, 2002, 307-311, 1057-1072.	2.7	187
152	Experimental simulation of the effect of transmuted helium on the mechanical properties of silicon carbide. Journal of Nuclear Materials, 2002, 307-311, 1141-1145.	2.7	31
153	The effect of high dose/high temperature irradiation on high purity fibers and their silicon carbide composites. Journal of Nuclear Materials, 2002, 307-311, 1157-1162.	2.7	65
154	Indentation fracture toughness of neutron irradiated silicon carbide. Journal of Nuclear Materials, 2002, 307-311, 1163-1167.	2.7	31
155	Analysis of possible deformation mechanisms in heliumâ€“ion irradiated SiC. Journal of Nuclear Materials, 2002, 307-311, 1178-1182.	2.7	3
156	Fabrication using a levitation melting method of Vâ€“4Crâ€“4Tiâ€“Siâ€“Alâ€“Y alloys and their mechanical properties. Journal of Nuclear Materials, 2002, 307-311, 555-559.	2.7	10
157	Effects of doping elements on oxidation properties of Vâ€“Crâ€“Ti type alloys in several environments. Journal of Nuclear Materials, 2002, 307-311, 601-604.	2.7	15
158	Effect of weld thermal cycle, stress and helium content on helium bubble formation in stainless steels. Journal of Nuclear Materials, 2002, 307-311, 327-330.	2.7	4
159	High resistance to helium embrittlement in reduced activation martensitic steels. Journal of Nuclear Materials, 2002, 307-311, 521-526.	2.7	47
160	Effect of simultaneous ion irradiation on microstructural change of SiC/SiC composites at high temperature. Journal of Nuclear Materials, 2002, 307-311, 1135-1140.	2.7	20
161	Mechanical property change and swelling behavior of SiC fiber after light-ion irradiation. Journal of Nuclear Materials, 2002, 307-311, 1152-1156.	2.7	7
162	Shear punch tests performed using a new low compliance test fixture. Journal of Nuclear Materials, 2002, 307-311, 1619-1623.	2.7	46

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163	Japanese Evaluated Nuclear Data Library Version 3 Revision-3: JENDL-3.3.. Journal of Nuclear Science and Technology, 2002, 39, 1125-1136.	1.3	335
164	Effect of Helium on Grain Boundary Segregation of Austenitic Stainless Steel After Proton Irradiation. Fusion Science and Technology, 2001, 39, 585-589.	0.6	3
165	Effects of Doping Elements on Oxidation Properties of Low-Activation Vanadium Alloys. Materials Transactions, 2001, 42, 1048-1051.	1.2	6
166	Surface Morphology Changes in a SiC/SiC Composite as Caused by Simultaneous Triple-Ion-Beam Irradiation. Materials Transactions, 2001, 42, 171-175.	1.2	5
167	Design and material issues for high performance SiCf/SiC-based fusion power cores. Fusion Engineering and Design, 2001, 55, 55-95.	1.9	172
168	Effect of helium implantation on mechanical properties and microstructure evolution of reduced-activation 9Crâ€“2W martensitic steel. Journal of Nuclear Materials, 2001, 299, 83-89.	2.7	24
169	Effect of weld thermal cycle and restraint stress on helium bubble formation in stainless steels. Journal of Nuclear Materials, 2000, 283-287, 1220-1223.	2.7	4
170	Effect of dual-beam-irradiation by helium and carbon ions on microstructure development of SiC/SiC composites. Journal of Nuclear Materials, 2000, 283-287, 268-272.	2.7	23
171	Microstructural development of neutron irradiated Wâ€“Re alloys. Journal of Nuclear Materials, 2000, 283-287, 1144-1147.	2.7	106
172	Microstructural changes of austenitic steels caused by proton irradiation under various conditions. Journal of Nuclear Materials, 2000, 283-287, 263-267.	2.7	4
173	Critical issues and current status of SiC/SiC composites for fusion. Journal of Nuclear Materials, 2000, 283-287, 128-137.	2.7	158
174	Study of helium effects in SiC/SiC composites under fusion reactor environment. Journal of Nuclear Materials, 2000, 283-287, 811-815.	2.7	14
175	Oxidation and hardness profile of Vâ€“Tiâ€“Crâ€“Siâ€“Alâ€“Y alloys. Journal of Nuclear Materials, 2000, 283-287, 1311-1315.	2.7	8
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