Akira Hasegawa

List of Publications by Year
in descending order

218 papers	7,403 citations	61984 43 h-index	62596 80 g-index
220 all docs	220 docs citations	220 times ranked	2806 citing authors

#	Article	IF	Citations
1	Japanese Evaluated Nuclear Data Library Version 3 Revision-3: JENDL-3.3. Journal of Nuclear Science and Technology, 2002, 39, 1125-1136.	1.3	469
2	Continuous SiC fiber, CVI SiC matrix composites for nuclear applications: Properties and irradiation effects. Journal of Nuclear Materials, 2014, 448, 448-476.	2.7	368
3	Japanese Evaluated Nuclear Data Library Version 3 Revision-3: JENDL-3.3.. Journal of Nuclear Science and Technology, 2002, 39, 1125-1136.	1.3	335
4	Current status and critical issues for development of SiC composites for fusion applications. Journal of Nuclear Materials, 2007, 367-370, 659-671.	2.7	310
5	Recent progress in R&D on tungsten alloys for divertor structural and plasma facing materials. Journal of Nuclear Materials, 2013, 442, S181-S189.	2.7	272
6	Promise and challenges of SiCf/SiC composites for fusion energy applications. Journal of Nuclear Materials, 2002, 307-311, 1057-1072.	2.7	187
7	Issues and advances in SiCf/SiC composites development for fusion reactors. Journal of Nuclear Materials, 2004, 329-333, 56-65.	2.7	178
8	Development of ultra-fine grained Wâ€“(0.25â€“0.8)wt%TiC and its superior resistance to neutron and 3MeV He-ion irradiations. Journal of Nuclear Materials, 2008, 377, 34-40.	2.7	178
9	Design and material issues for high performance SiCf/SiC-based fusion power cores. Fusion Engineering and Design, 2001, 55, 55-95.	1.9	172
10	Neutron irradiation effects on tungsten materials. Fusion Engineering and Design, 2014, 89, 1568-1572.	1.9	167
11	Critical issues and current status of SiC/SiC composites for fusion. Journal of Nuclear Materials, 2000, 283-287, 128-137.	2.7	158
12	Neutron irradiation effects on the microstructural development of tungsten and tungsten alloys. Journal of Nuclear Materials, 2016, 471, 175-183.	2.7	151
13	Baseline high heat flux and plasma facing materials for fusion. Nuclear Fusion, 2017, 57, 092006.	3.5	141
14	Behavior of tungsten under irradiation and plasma interaction. Journal of Nuclear Materials, 2019, 519, 334-368.	2.7	129
15	Recent advances and issues in development of silicon carbide composites for fusion applications. Journal of Nuclear Materials, 2009, 386-388, 622-627.	2.7	124
16	Effects of Transmutation Elements on Neutron Irradiation Hardening of Tungsten. Materials Transactions, 2007, 48, 2399-2402.	1.2	120
17	Microstructural development of tungsten and tungstenâ€“rhenium alloys due to neutron irradiation in HFIR. Journal of Nuclear Materials, 2014, 455, 460-463.	2.7	118
18	Microstructure Development in Neutron Irradiated Tungsten Alloys. Materials Transactions, 2011, 52, 1447-1451.	1.2	112

#	Article	IF	Citations
19	Microstructural development of neutron irradiated Wâ€“Re alloys. Journal of Nuclear Materials, 2000, 283-287, 1144-1147.	2.7	106
20	Property change mechanism in tungsten under neutron irradiation in various reactors. Journal of Nuclear Materials, 2011, 417, 491-494.	2.7	99
21	Neutron Irradiation Behavior of Tungsten. Materials Transactions, 2013, 54, 466-471.	1.2	96
22	Property change of advanced tungsten alloys due to neutron irradiation. Journal of Nuclear Materials, 2013, 442, S273-S276.	2.7	95
23	Neutron-Induced Fission of²³³U,²³⁸U,²³²Th,²³⁹Pu,²³⁷Np,^{nat}Pb and²⁰⁹Bi Relative to²³⁵U in the Energy Range 1-200 MeV. Journal of Nuclear Science and Technology, 2002, 39, 230-233.	1.3	94
24	Effects of transmutation elements on the microstructural evolution and electrical resistivity of neutron-irradiated tungsten. Journal of Nuclear Materials, 2009, 386-388, 218-221.	2.7	85
25	Effects of Re Content and Fabrication Process on Microstructural Changes and Hardening in Neutron Irradiated Tungsten. Materials Transactions, 2012, 53, 2145-2150.	1.2	84
26	Stability and mobility of rhenium and osmium in tungsten: first principles study. Modelling and Simulation in Materials Science and Engineering, 2014, 22, 075006.	2.0	81
27	Recrystallization behavior of hot-rolled pure tungsten and its alloy plates during high-temperature annealing. Nuclear Materials and Energy, 2018, 15, 158-163.	1.3	71
28	Evaluation of irradiation hardening of proton irradiated stainless steels by nanoindentation. Journal of Nuclear Materials, 2014, 446, 142-147.	2.7	70
29	The effect of high dose/high temperature irradiation on high purity fibers and their silicon carbide composites. Journal of Nuclear Materials, 2002, 307-311, 1157-1162.	2.7	65
30	Microstructural development and irradiation hardening of W and Wâ€“(3â€“26) wt%Re alloys after high-temperature neutron irradiation to 0.15â€‰dpa. Nuclear Fusion, 2006, 46, 877-883.	3.5	64
31	Neutron energy spectrum influence on irradiation hardening and microstructural development of tungsten. Journal of Nuclear Materials, 2016, 479, 249-254.	2.7	64
32	Thermal properties of pure tungsten and its alloys for fusion applications. Fusion Engineering and Design, 2018, 132, 1-6.	1.9	64
33	Precipitation of Solid Transmutation Elements in Irradiated Tungsten Alloys. Materials Transactions, 2008, 49, 2259-2264.	1.2	62
34	Overview of design and R&D of test blankets in Japan. Fusion Engineering and Design, 2006, 81, 415-424.	1.9	61
35	Induced radioactivity of component materials by 16-MeV protons and 30-MeV alpha particles. Journal of Nuclear Materials, 1984, 123, 972-976.	2.7	57
36	Mechanical properties of tungsten: Recent research on modified tungsten materials in Japan. Journal of Nuclear Materials, 2021, 543, 152506.	2.7	55

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37	Helium-bubble formation behavior of SiCf/SiC composites after helium implantation. Journal of Nuclear Materials, 1999, 264, 355-358.	2.7	48
38	High resistance to helium embrittlement in reduced activation martensitic steels. Journal of Nuclear Materials, 2002, 307-311, 521-526.	2.7	47
39	Effect of neutron irradiation on rhenium cluster formation in tungsten and tungsten-rhenium alloys. Journal of Nuclear Materials, 2018, 507, 78-86.	2.7	47
40	Shear punch tests performed using a new low compliance test fixture. Journal of Nuclear Materials, 2002, 307-311, 1619-1623.	2.7	46
41	Migration of rhenium and osmium interstitials in tungsten. Journal of Nuclear Materials, 2015, 467, 418-423.	2.7	46
42	Effect of self-ion irradiation on hardening and microstructure of tungsten. Nuclear Materials and Energy, 2016, 9, 430-435.	1.3	44
43	Annealing behavior of irradiation hardening and microstructure in helium-implanted reduced activation martensitic steel. Journal of Nuclear Materials, 2000, 283-287, 827-831.	2.7	43
44	Mechanical properties of advanced SiC fiber composites irradiated at very high temperatures. Journal of Nuclear Materials, 2011, 417, 416-420.	2.7	43
45	Microstructural development and radiation hardening of neutron irradiated Moâ€“Re alloys. Journal of Nuclear Materials, 2004, 324, 62-70.	2.7	41
46	Effects of helium on ductile-brittle transition behavior of reduced-activation ferritic steels after high-concentration helium implantation at high temperature. Journal of Nuclear Materials, 2009, 386-388, 241-244.	2.7	41
47	Recent progress of tungsten R&D for fusion application in Japan. Physica Scripta, 2011, T145, 014029.	2.5	39
48	Synergistic effects of implanted helium and hydrogen and the effect of irradiation temperature on the microstructure of SiC/SiC composites. Journal of Nuclear Materials, 2004, 335, 508-514.	2.7	38
49	Effects of tool rotation speed on the mechanical properties and microstructure of friction stir welded ODS steel. Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing, 2014, 595, 291-296.	5.6	38
50	Tensile behavior of helium charged Vî—¸Tiî—¸Crî—¸Si type alloys. Journal of Nuclear Materials, 1996, 233-237, 447-451.	2.7	35
51	Anisotropy in the Mechanical Properties of Potassium and Rhenium Doped Tungsten Alloy Plates for Fusion Reactor Applications. Fusion Science and Technology, 2015, 68, 690-693.	1.1	35
52	Silicon Carbide and Silicon Carbide Composites for Fusion Reactor Application. Materials Transactions, 2013, 54, 472-476.	1.2	34
53	Microstructure development of dispersion-strengthened tungsten due to neutron irradiation. Journal of Nuclear Materials, 2014, 449, 213-218.	2.7	34
54	Effects of temperature and strain rate on the tensile properties of potassium-doped tungsten. Journal of Nuclear Materials, 2015, 461, 357-364.	2.7	33

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55	Tungsten modified by potassium doping and rhenium addition for fusion reactor applications. Fusion Engineering and Design, 2020, 152, 111445.	1.9	33
56	Experimental simulation of the effect of transmuted helium on the mechanical properties of silicon carbide. Journal of Nuclear Materials, 2002, 307-311, 1141-1145.	2.7	31
57	Indentation fracture toughness of neutron irradiated silicon carbide. Journal of Nuclear Materials, 2002, 307-311, 1163-1167.	2.7	31
58	R&D of A MW-class solid-target for a spallation neutron source. Journal of Nuclear Materials, 2003, 318, 38-55.	2.7	31
59	Effects of transmutation elements on the defect structure development of W irradiated by protons and neutrons. Journal of Nuclear Materials, 2008, 377, 348-351.	2.7	30
60	A review of impact properties of tungsten materials. Fusion Engineering and Design, 2018, 135, 196-203.	1.9	30
61	Mechanical properties and microstructures of high-chromium Vâ€“Crâ€“Ti type alloys. Journal of Nuclear Materials, 2004, 329-333, 457-461.	2.7	29
62	Suppression of radiation-induced point defects by rhenium and osmium interstitials in tungsten. Scientific Reports, 2016, 6, 36738.	3.3	28
63	Degradation of tungsten monoblock divertor under cyclic high heat flux loading. Fusion Engineering and Design, 2017, 120, 49-60.	1.9	28
64	PHENIX U.S.-Japan Collaboration Investigation of Thermal and Mechanical Properties of Thermal Neutronâ€“Shielded Irradiated Tungsten. Fusion Science and Technology, 2019, 75, 499-509.	1.1	28
65	Tensile and impact properties of tungsten-rhenium alloy for plasma-facing components in fusion reactor. Fusion Engineering and Design, 2019, 148, 111323.	1.9	27
66	Strain rate dependence of tensile properties of tungsten alloys for plasma-facing components in fusion reactors. Fusion Engineering and Design, 2016, 109-111, 1674-1677.	1.9	26
67	Effect of helium implantation on mechanical properties and microstructure evolution of reduced-activation 9Crâ€“2W martensitic steel. Journal of Nuclear Materials, 2001, 299, 83-89.	2.7	24
68	Tensile properties of K-doped Wâ€“3%Re. Fusion Engineering and Design, 2014, 89, 1033-1036.	1.9	24
69	Improvement of impact properties of tungsten by potassium doping. Fusion Engineering and Design, 2019, 140, 48-61.	1.9	24
70	Tensile behavior and microstructure of neutron-irradiated Mo-5% Re alloy. Journal of Nuclear Materials, 1995, 225, 259-266.	2.7	23
71	Effect of dual-beam-irradiation by helium and carbon ions on microstructure development of SiC/SiC composites. Journal of Nuclear Materials, 2000, 283-287, 268-272.	2.7	23
72	Study on electron beam weld joints between pure vanadium and SUS316L stainless steel. Journal of Nuclear Materials, 2013, 442, S562-S566.	2.7	23

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73	Development of fatigue life evaluation method using small specimen. Journal of Nuclear Materials, 2013, 441, 125-132.	2.7	23
74	Neutron irradiation effects on the mechanical properties of powder metallurgical processed tungsten alloys. Journal of Nuclear Materials, 2020, 529, 151910.	2.7	23
75	Effects of helium on mechanical properties of tungsten for fusion applications. Nuclear Materials and Energy, 2018, 15, 154-157.	1.3	22
76	Neutron irradiation tolerance of potassium-doped and rhenium-alloyed tungsten. Journal of Nuclear Materials, 2021, 553, 153009.	2.7	21
77	Neutron irradiation-enhanced grain growth in tungsten and tungsten alloys. Journal of Alloys and Compounds, 2022, 901, 163419.	5.5	21
78	Neutron irradiation embrittlement of molybdenum rhenium alloys and their improvement by heat treatment. Journal of Nuclear Materials, 1998, 258-263, 902-906.	2.7	20
79	Effect of simultaneous ion irradiation on microstructural change of SiC/SiC composites at high temperature. Journal of Nuclear Materials, 2002, 307-311, 1135-1140.	2.7	20
80	Influence of heat-treatment on tensile behavior of neutron irradiated molybdenum. Journal of Nuclear Materials, 1996, 233-237, 565-569.	2.7	19
81	Study of hydrogen effects on microstructural development of SiC base materials under simultaneous irradiation with He- and Si-ion irradiation conditions. Journal of Nuclear Materials, 2004, 329-333, 582-586.	2.7	19
82	Effect of microstructural anisotropy on the mechanical properties of K-doped tungsten rods for plasma facing components. Fusion Engineering and Design, 2016, 109-111, 1549-1553.	1.9	19
83	Analysis and measurement of residual stress distribution of vanadium/ceramics joints for fusion reactor applications. Journal of Nuclear Materials, 1998, 258-263, 1517-1522.	2.7	18
84	JENDL-3.2 Covariance File. Journal of Nuclear Science and Technology, 2002, 39, 40-43.	1.3	18
85	Fabrication and Characterization of W-Re-Os Alloys for Studying Transmutation Effects of W in Fusion Reactors. Materials Transactions, 2004, 45, 2657-2660.	1.2	18
86	First-principles study of solvent-solute mixed dumbbells in body-centered-cubic tungsten crystals. Journal of Nuclear Materials, 2018, 505, 15-21.	2.7	18
87	Influence of Cr, Ti concentrations on oxidation and corrosion resistance of Vâ€"Crâ€"Ti type alloys. Journal of Nuclear Materials, 2004, 329-333, 452-456.	2.7	17
88	Oxidation behavior of SiC/SiC composites for helium cooled solid breeder blanket. Fusion Engineering and Design, 2008, 83, 1490-1494.	1.9	17
89	Overview on recent progress toward small specimen test technique. Fusion Engineering and Design, 2015, 98-99, 2089-2093.	1.9	17
90	Tensile and fatigue properties of potassium doped and rhenium containing tungsten rods for fusion reactor applications. Fusion Engineering and Design, 2016, 109-111, 1538-1542.	1.9	17

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91	Feasibility of Utilizing Tungsten Rod for Fusion Reactor Divertor. Fusion Science and Technology, 2017, 72, 673-679.	1.1	17
92	Helium implantation effects on low activation 9Cr martensitic steels. Journal of Nuclear Materials, 1992, 191-194, 910-914.	2.7	16
93	High-Temperature Helium Embrittlement of 316FR Steel. Journal of Nuclear Science and Technology, 2011, 48, 130-134.	1.3	16
94	Tensile properties of powder-metallurgical-processed tungsten alloys after neutron irradiation near recrystallization temperatures. Journal of Nuclear Materials, 2020, 542, 152505.	2.7	16
95	Effects of doping elements on oxidation properties of Vâ€“Crâ€“Ti type alloys in several environments. Journal of Nuclear Materials, 2002, 307-311, 601-604.	2.7	15
96	Development of advanced blanket performance under irradiation and system integration through JUPITER-II project. Fusion Engineering and Design, 2008, 83, 842-849.	1.9	15
97	Effect of Specimen Shape on the Low Cycle Fatigue Life of Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 47-52.	1.3	15
98	Analysis of the temperature and thermal stress in pure tungsten monoblock during heat loading and the influences of alloying and dispersion strengthening on these responses. Fusion Engineering and Design, 2016, 107, 44-50.	1.9	15
99	The effect of voids on the hardening of body-centered cubic Fe. Journal of Nuclear Materials, 2016, 471, 233-238.	2.7	15
100	Tungstenâ€“tantalum alloys for fusion reactor applications. Journal of Nuclear Materials, 2022, 566, 153740.	2.7	15
101	Tensile properties of a series of Vâ€“4Tiâ€“4Cr alloys containing small amounts of Si, Al and Y, and the influence of helium implantation. Journal of Nuclear Materials, 1998, 258-263, 1497-1501.	2.7	14
102	Study of helium effects in SiC/SiC composites under fusion reactor environment. Journal of Nuclear Materials, 2000, 283-287, 811-815.	2.7	14
103	Present status of study on development of materials resistant to radiation and beam impact. Journal of Nuclear Materials, 2008, 377, 21-27.	2.7	14
104	Depth dependence of radiation hardening in 10 MeV 4He+-ION bombarded molybdenum. Journal of Nuclear Materials, 1981, 104, 1169-1173.	2.7	13
105	Effect of helium implantation at 533â€“573 K on tensile properties of 9Cr martensitic steels. Journal of Nuclear Materials, 1993, 202, 266-274.	2.7	13
106	Effects of small amount of additional elements on control of interstitial impurities and mechanical properties of Vâ€“4Crâ€“4Tiâ€“Siâ€“Alâ€“Y alloys. Journal of Nuclear Materials, 2004, 326, 1-8.	2.7	13
107	Thermal diffusivity of irradiated tungsten and tungsten-rhenium alloys. Journal of Nuclear Materials, 2021, 543, 152594.	2.7	13
108	Behavior of helium gas atoms and bubbles in low activation 9Cr martensitic steels. Journal of Nuclear Materials, 1994, 212-215, 720-724.	2.7	12

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109	Study on irradiation induced corrosion behavior in austenitic stainless steel using hydrogen-ion bombardment. Journal of Nuclear Materials, 2004, 329-333, 652-656.	2.7	12
110	Cavity Formation in a SiC/SiC Composite under Simultaneous Irradiation of Hydrogen, Helium and Silicon Ions. Materials Transactions, 2005, 46, 536-542.	1.2	12
111	Improvement of Surface Exfoliation Behavior by Helium-ion Bombardment of a Tungsten Alloy Fabricated by Mechanical Alloying. Journal of Nuclear Science and Technology, 2009, 46, 717-723.	1.3	12
112	Study on Fatigue Life Evaluation Using Small Specimens for Testing Neutron-Irradiated Materials. Journal of Nuclear Science and Technology, 2011, 48, 60-64.	1.3	12
113	Effect of post-weld heat treatment and neutron irradiation on a dissimilar-metal joint between F82H steel and 316L stainless steel. Fusion Engineering and Design, 2015, 98-99, 1968-1972.	1.9	12
114	Nb tube processed Nb/sub 3/Al multifilamentary superconductors. IEEE Transactions on Magnetics, 1991, 27, 2045-2048.	2.1	11
115	Effect of irradiation on the microstructure of Nicalon fibers. Journal of Nuclear Materials, 1996, 231, 245-248.	2.7	11
116	Rapid oxidation and its effects on mechanical properties of Vâ€“Tiâ€“Crâ€“Si type alloys. Journal of Nuclear Materials, 1998, 258-263, 1507-1511.	2.7	11
117	Effect of He pre-implantation and neutron irradiation on mechanical properties of SiC/SiC composite. Journal of Nuclear Materials, 2004, 329-333, 577-581.	2.7	11
118	Mechanical Property Changes and Irradiation Hardening Due to Dissimilar Metal Welding with Reduced Activation Ferritic/Martensitic Steel and 316L Stainless Steel. Fusion Science and Technology, 2009, 56, 318-322.	1.1	11
119	Irradiation damages of structural materials under different irradiation environments. Journal of Nuclear Materials, 2021, 543, 152503.	2.7	11
120	Helium embrittlement in Fe-Ni-Cr austenitic alloys and ferritic MA 956 alloy. Journal of Nuclear Materials, 1989, 169, 198-205.	2.7	10
121	Fabrication using a levitation melting method of Vâ€“4Crâ€“4Tiâ€“Siâ€“Alâ€“Y alloys and their mechanical properties. Journal of Nuclear Materials, 2002, 307-311, 555-559.	2.7	10
122	Synergistic Effect of Displacement Damage, Helium and Hydrogen of Silicon Carbide Composite. Fusion Science and Technology, 2003, 44, 175-180.	1.1	10
123	Study on Compatibility Between Silicon Carbide and Solid Breeding Materials Under Neutron Irradiation. Fusion Science and Technology, 2011, 60, 288-291.	1.1	10
124	Effect of specimen shape on micro-crack growth behavior under fatigue in reduced activation ferritic/martensitic steel. Journal of Nuclear Materials, 2011, 417, 131-134.	2.7	10
125	Tungsten as a Plasma-Facing Material. , 2020, , 19-53.		10
126	Microstructure and Superconducting Properties of Nb₃Al Multifilamentary Wires Processed by Nb-Tube Method. Nippon Kinzoku Gakkaishi/Journal of the Japan Institute of Metals, 1991, 55, 472-480.	0.4	9

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127	Helium implantation effects on mechanical properties of SiCf/SiC composites. Journal of Nuclear Materials, 1998, 253, 31-35.	2.7	9
128	Compatibility between SiC and Li ceramics for solid breeding blanket system. Journal of Nuclear Materials, 2009, 386-388, 628-630.	2.7	9
129	Effect of He implantation on fracture behavior and microstructural evolution in F82H. Journal of Nuclear Materials, 2014, 455, 690-694.	2.7	9
130	Microchemical and microstructural changes of austenitic steels caused by proton irradiation following helium implantation. Journal of Nuclear Materials, 1998, 258-263, 1694-1699.	2.7	8
131	Weldability of helium-containing stainless steels using a YAG laser. Journal of Nuclear Materials, 1998, 258-263, 2018-2022.	2.7	8
132	Oxidation and hardness profile of Vâ€“Tiâ€“Crâ€“Siâ€“Alâ€“Y alloys. Journal of Nuclear Materials, 2000, 283-287, 1311-1315.	2.7	8
133	Effect of PWHT on the Mechanical and Metallographical Properties of a Dissimilar-Metal Weld Joint of F82H and SUS316L Steels. Fusion Science and Technology, 2011, 60, 334-338.	1.1	8
134	Study on stress relaxation behavior of silicon carbide by BSR method. Journal of Nuclear Materials, 2011, 417, 356-358.	2.7	8
135	Effect of Grain Structure Anisotropy and Recrystallization on Tensile Properties of Swaged Tungsten Rod. Plasma and Fusion Research, 2015, 10, 1405073-1405073.	0.7	8
136	Solid state diffusion bonding of doped tungsten alloys with different thermo-mechanical properties. Fusion Engineering and Design, 2018, 136, 76-81.	1.9	8
137	Helium effects on recovery and recrystallization of powder metallurgically processed tungsten. Physica Scripta, 2020, T171, 014016.	2.5	8
138	Mechanical property change and swelling behavior of SiC fiber after light-ion irradiation. Journal of Nuclear Materials, 2002, 307-311, 1152-1156.	2.7	7
139	Evaluation of interface strength between metal and ceramics to be utilized for development of fusion reactor components. Journal of Nuclear Materials, 2009, 386-388, 689-691.	2.7	7
140	Midterm Summary of Japan-US Fusion Cooperation Program TITAN. Fusion Science and Technology, 2011, 60, 321-328.	1.1	7
141	Improved structural strength and lifetime of monoblock divertor targets by using doped tungsten alloys under cyclic high heat flux loading. Physica Scripta, 2017, T170, 014011.	2.5	7
142	Effects of Doping Elements on Oxidation Properties of Low-Activation Vanadium Alloys. Materials Transactions, 2001, 42, 1048-1051.	1.2	6
143	Effect of displacement damage up to 50dpa on microstructural development in SiC/SiC composites. Journal of Nuclear Materials, 2007, 367-370, 698-702.	2.7	6
144	Fatigue Life Assessment Based on Crack Growth Behavior in Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 457-461.	1.3	6

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145	Effect of helium on fatigue crack growth and life of reduced activation ferritic/martensitic steel. Journal of Nuclear Materials, 2013, 442, S43-S47.	2.7	6
146	Japanese activities of the R&D on silicon carbide composites in the broader approach period and beyond. Journal of Nuclear Materials, 2018, 511, 582-590.	2.7	6
147	Microhardness testing of molybdenum irradiated by He ions. Journal of Nuclear Materials, 1985, 133-134, 657-661.	2.7	5
148	Hydrogen absorption behavior of molybdenum under glow discharge. Journal of Nuclear Materials, 1986, 141-143, 234-237.	2.7	5
149	Application of micro-indentation to irradiated alumina and vanadium/alumina joints. Journal of Nuclear Materials, 1996, 233-237, 1279-1283.	2.7	5
150	Surface Morphology Changes in a SiC/SiC Composite as Caused by Simultaneous Triple-Ion-Beam Irradiation. Materials Transactions, 2001, 42, 171-175.	1.2	5
151	Development of Low-Activation Design Method for Reduction of Radioactive Waste Below Clearance Level. , 2008, , .		5
152	Distribution of Cobalt between MgO-Saturated FeO<I>_x</I>-MgO-CaO-SiO₂ Slag and Fe-Cu-Co Molten Alloy. Materials Transactions, 2008, 49, 2636-2641.	1.2	5
153	Indentation Properties of Silicon Carbide after Neutron Irradiation and Helium Implantation. IOP Conference Series: Materials Science and Engineering, 2011, 18, 162007.	0.6	5
154	Effect of heat treatment on bend stress relaxation of pure tungsten. Fusion Engineering and Design, 2013, 88, 1735-1738.	1.9	5
155	Void Swelling of Proton Irradiated Fe-15Cr-20Ni Ternary Alloy. Journal of Nuclear Science and Technology, 1996, 33, 239-244.	1.3	4
156	Effect of weld thermal cycle and restraint stress on helium bubble formation in stainless steels. Journal of Nuclear Materials, 2000, 283-287, 1220-1223.	2.7	4
157	Microstructural changes of austenitic steels caused by proton irradiation under various conditions. Journal of Nuclear Materials, 2000, 283-287, 263-267.	2.7	4
158	Integral Test of JENDL-3.3 with Shielding Benchmarks. Journal of Nuclear Science and Technology, 2002, 39, 841-846.	1.3	4
159	Neutron Scattering on238U and232Th. Journal of Nuclear Science and Technology, 2002, 39, 148-151.	1.3	4
160	Effect of weld thermal cycle, stress and helium content on helium bubble formation in stainless steels. Journal of Nuclear Materials, 2002, 307-311, 327-330.	2.7	4
161	Perspectives of SiC-Based Ceramic Composites and Their Applications to Fusion Reactors 4.Neutron Irradiation Effects in SiC and SiC⁄SiC Composites. Journal of Plasma and Fusion Research, 2004, 80, 24-30.	0.4	4
162	Plan and Strategy for ITER Blanket Testing in Japan. Fusion Science and Technology, 2005, 47, 1023-1030.	1.1	4

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163	Evaluation of bonding strength between yttria coating and vanadium alloys for development of self-cooled blanket. Journal of Nuclear Materials, 2011, 417, 1253-1256.	2.7	4
164	Tritium trapping in silicon carbide in contact with solid breeder under high flux isotope reactor irradiation. Journal of Nuclear Materials, 2013, 442, S497-S500.	2.7	4
165	High Temperature Fatigue Life Evaluation Using Small Specimen. Plasma and Fusion Research, 2017, 12, 1405022-1405022.	0.7	4
166	Fatigue properties of ferritic/martensitic steel after neutron irradiation and helium implantation. Nuclear Materials and Energy, 2020, 24, 100764.	1.3	4
167	Neutron Resonance Heterogeneity Effect in the Vicinity of a Gross Region Interface. Nuclear Science and Engineering, 1977, 64, 786-791.	1.1	3
168	Helium effects in iron- and nickel-base developmental alloys. Journal of Nuclear Materials, 1988, 155-157, 1049-1053.	2.7	3
169	Benchmark Tests of Gamma-Ray Production Data in JENDL-3 for Some Important Nuclides. Journal of Nuclear Science and Technology, 1990, 27, 844-852.	1.3	3
170	Interface strength of SiC/SiC composites with and without helium implantation using micro-indentation test. Journal of Nuclear Materials, 1998, 258-263, 1562-1566.	2.7	3
171	Deformation analysis of small size bend specimens by FEM calculation to estimate irradiation induced embrittlement of Mo and W. Journal of Nuclear Materials, 1998, 258-263, 466-470.	2.7	3
172	Effect of Helium on Grain Boundary Segregation of Austenitic Stainless Steel After Proton Irradiation. Fusion Science and Technology, 2001, 39, 585-589.	0.6	3
173	Analysis of possible deformation mechanisms in heliumâ€“ion irradiated SiC. Journal of Nuclear Materials, 2002, 307-311, 1178-1182.	2.7	3
174	Fatigue Crack Initiation in Proton-Irradiated Austenitic Stainless Steel. Journal of Nuclear Science and Technology, 2011, 48, 1265-1271.	1.3	3
175	Study on Dissimilar-Material Welding with Vanadium and Austenitic Stainless Steel. Fusion Science and Technology, 2011, 60, 417-421.	1.1	3
176	Helium effects on the tensile property of 316FR stainless steel at 650 and 750 Â°C. Journal of Nuclear Materials, 2011, 417, 1030-1033.	2.7	3
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