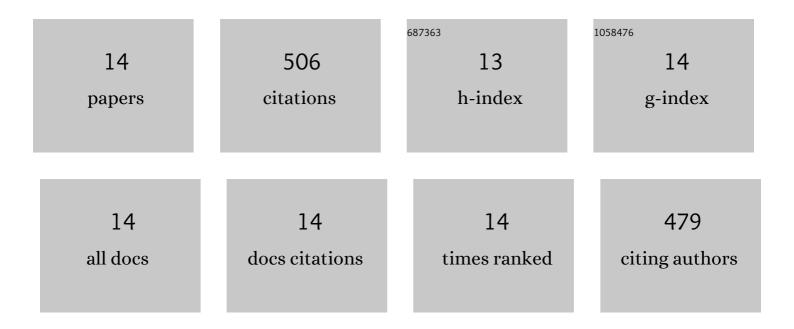


List of Publications by Year in descending order

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#	Article	IF	CITATIONS
1	Phase and defect evolution in uranium-nitrogen-oxygen system under irradiation. Acta Materialia, 2021, 208, 116778.	7.9	21
2	Purification effect of the methods used for the preparation of the ultra-high purity graphite. Diamond and Related Materials, 2021, 120, 108704.	3.9	12
3	Zr alloy protection against high-temperature oxidation: Coating by a double-layered structure with active and passive functional properties. Corrosion Science, 2020, 163, 108270.	6.6	15
4	Unveiling hermetic failure of ceramic tubes by digital image correlation and acoustic emission. Journal of the American Ceramic Society, 2020, 103, 2146-2159.	3.8	13
5	Inhibition effect and mechanism of polyacrylamide for steel corrosion in simulated concrete pore solution. Construction and Building Materials, 2020, 259, 120425.	7.2	34
6	Influence of sulfate salt type on passive film of steel in simulated concrete pore solution. Construction and Building Materials, 2019, 223, 352-359.	7.2	32
7	Development of cold spray chromium coatings for improved accident tolerant zirconium-alloy cladding. Journal of Nuclear Materials, 2019, 519, 247-254.	2.7	79
8	Evaluation of steam corrosion and water quenching behavior of zirconium-silicide coated LWR fuel claddings. Journal of Nuclear Materials, 2018, 499, 256-267.	2.7	54
9	Development of Cold Spray Coatings for Accident-Tolerant Fuel Cladding in Light Water Reactors. Jom, 2018, 70, 198-202.	1.9	52
10	Magnetron sputter deposition of zirconium-silicide coating for mitigating high temperature oxidation of zirconium-alloy. Surface and Coatings Technology, 2017, 316, 30-38.	4.8	40
11	Quantifying the three-dimensional damage and stress redistribution mechanisms of braided SiC/SiC composites by in situ volumetric digital image correlation. Scripta Materialia, 2017, 130, 238-241.	5.2	37
12	Nanocrystalline diamond protects Zr cladding surface against oxygen and hydrogen uptake: Nuclear fuel durability enhancement. Scientific Reports, 2017, 7, 6469.	3.3	16
13	Synthesis and sintering of UN-UO2 fuel composites. Journal of Nuclear Materials, 2015, 466, 745-754.	2.7	48
14	Characterization of structural defects in nuclear graphite IG-110 and NBG-18. Journal of Nuclear Materials, 2014, 446, 193-199.	2.7	53