Nasir M Mirza

List of Publications by Year
in descending order

		623574	752573
41 papers	465 citations	14 h-index	20 g-index
41 all docs	41 docs citations	41 times ranked	222 citing authors

#	Article	IF	Citations
1	Differential evolution based computation intelligence solver for elliptic partial differential equations. Frontiers of Information Technology and Electronic Engineering, 2019, 20, 1445-1456.	1.5	14
2	Parametric study of iodine-129 releases from nuclear fuel to fuel-clad gap & primary coolant in PWRs. Annals of Nuclear Energy, 2019, 128, 181-189.	0.9	2
3	Bio-inspired heuristics for layer thickness optimization in multilayer piezoelectric transducer for broadband structures. Soft Computing, 2019, 23, 3449-3463.	2.1	36
4	Biologically inspired computing framework for solving two-point boundary value problems using differential evolution. Neural Computing and Applications, 2017, 28, 2165-2179.	3.2	14
5	Effect of Kinetic Parameters on Simultaneous Ramp Reactivity Insertion Plus Beam Tube Flooding Accident in a Typical Low Enriched U_3Si_2-Al Fuel-Based Material Testing Reactor-Type Research Reactor. Nuclear Engineering and Technology, 2017, 49, 700-709.	1.1	5
6	Parametric Study of Time-Dependent Corrosion Product Activity due to ^{56}Mn, ^{58}Co, and ^{60}Co in the Primary Coolant Circuit of a Typical Pressurized Water Reactor. Journal of Chemistry, 2015, 2015, 1-10.	0.9	2
7	Simultaneous multiple reactivity insertions in a typical MTR-type research reactor having U3Si2–Al fuel. Annals of Nuclear Energy, 2015, 85, 869-878.	0.9	5
8	Effect of high density dispersion fuels on transient behavior of MTR type research reactor under multiple reactivity transients. Progress in Nuclear Energy, 2015, 85, 511-517.	1.3	4
9	Determination of age specific 131I S-factor values for thyroid using anthropomorphic phantom in geant4 simulations. Applied Radiation and Isotopes, 2014, 90, 15-22.	0.7	2
10	Core loading pattern optimization of a typical two-loop 300MWe PWR using Simulated Annealing (SA), novel crossover Genetic Algorithms (GA) and hybrid GA(SA) schemes. Annals of Nuclear Energy, 2014, 65, 122-131.	0.9	61
11	Study of successive ramp reactivity insertions in typical pool-type research reactors. Progress in Nuclear Energy, 2013, 66, 115-123.	1.3	4
12	Comparative study of MIRD, experimental and GEANT4 simulations for uniformly distributed I-131 in cylindrical and spherical thyroid models. Radiation Measurements, 2012, 47, 406-409.	0.7	2
13	Kinetic study of fission product activity released inside containment under loss of coolant transients in a typical MTR system. Applied Radiation and Isotopes, 2012, 70, 2711-2719.	0.7	11
14	Absorbed dose estimations of 131 I for critical organs using the GEANT4 Monte Carlo simulation code. Chinese Physics C, 2012, 36, 1150-1156.	1.5	2
15	Analysis of core life-time and neutronic parameters for HEU and potential LEU/MEU fuels in a typical MNSR. Annals of Nuclear Energy, 2012, 47, 46-52.	0.9	7
16	Simulation of corrosion product activity in extended operating cycles of PWRs under flow rate transient and nonlinearly rising corrosion rates coupled with pH effects. Nuclear Engineering and Design, 2012, 249, 388-399.	0.8	3
17	Time-dependent corrosion product activity in a typical PWR due to changes in coolant chemistry for long-term fuel cycles. Progress in Nuclear Energy, 2012, 58, 100-107.	1.3	9
18	Response to: Comment on the paper "Post-Shutdown decay power and radionuclide inventories in the discharged fuels of HEU and potential LEU miniature neutron source reactors" by Mirza, S.M., Khan, A., Mirza N.M. [Ann. Nucl. Energy 37 (2010) 701–706]. Annals of Nuclear Energy, 2011, 38, 2865-2866.	0.9	0

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19	Sensitivity analysis of fission product activity in primary coolant of typical PWRs. Progress in Nuclear Energy, 2011, 53, 245-249.	1.3	10
20	Static and dynamic sensitivity analysis of corrosion product activity in primary coolant circuits of pressurized water reactors. Progress in Nuclear Energy, 2010, 52, 648-654.	1.3	6
21	Source term evaluation for the upgraded LEU Pakistan Research Reactor-1 under severe accidents. Nuclear Engineering and Design, 2010, 240, 3740-3750.	0.8	15
22	Post-shutdown decay power and radionuclide inventories in the discharged fuels of HEU and potential LEU miniature neutron source reactors. Annals of Nuclear Energy, 2010, 37, 701-706.	0.9	10
23	Comparative study of actinide and fission product inventory of HEU and potential LEU fuels for MNSRs. Progress in Nuclear Energy, 2009, 51, 129-134.	1.3	13
24	Modeling and simulation of release of radioactivity from a typical MTR type research reactor under accidental conditions. , 2009, , .		0
25	Stochastic simulation of fission product activity in primary coolant due to fuel rod failures in typical PWRs under power transients. Journal of Nuclear Materials, 2008, 372, 132-140.	1.3	9
26	Two-group, three-dimensional model based study of reactivity induced transients in upgraded LEU material test reactors. Annals of Nuclear Energy, 2008, 35, 647-655.	0.9	6
27	A comparative neutronic study of the standard HEU core and various potential LEU alternatives for a typical MNSR system. Nuclear Engineering and Design, 2008, 238, 2302-2307.	0.8	11
28	Kinetic simulation of fission product activity in primary coolant of typical PWRs under power perturbations. Nuclear Engineering and Design, 2007, 237, 199-205.	0.8	19
29	Effect of flow rate transients on fission product activity in primary coolant of PWRs. Progress in Nuclear Energy, 2007, 49, 120-129.	1.3	10
30	Kinetic study of corrosion product activity in primary coolant pipes of a typical PWR under flow rate transients and linearly increasing corrosion rates. Journal of Nuclear Materials, 2005, 346, 282-292.	1.3	14
31	Simulation of corrosion product activity for nonlinearly rising corrosion on inner surfaces of primary coolant pipes of a typical PWR under flow rate transients. Applied Radiation and Isotopes, 2005, 62, 681-692.	0.7	14
32	Computer simulation of corrosion product activity in primary coolants of a typical PWR under flow rate transients and linearly accelerating corrosion. Annals of Nuclear Energy, 2003, 30, 831-851.	0.9	14
33	Experimental study of effect of void volume fraction on neutron diffusion parameters in water. Radiation Physics and Chemistry, 2002, 64, 349-357.	1.4	0
34	Modeling and simulation of corrosion product activity in pressurized water reactors under power perturbations. Annals of Nuclear Energy, 1999, 26, 561-578.	0.9	17
35	Sensitivity of reactivity insertion limits with respect to safety parameters in a typical MTR. Annals of Nuclear Energy, 1999, 26, 1517-1535.	0.9	31
36	Simulation of corrosion product activity in pressurized water reactors under flow rate transients. Annals of Nuclear Energy, 1998, 25, 331-345.	0.9	17

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37	Simulation of reactivity transients in current MTRs. Annals of Nuclear Energy, 1998, 25, 1465-1484.	0.9	30
38	Study of the void coefficients of reactivity in a typical pool type research reactor. Annals of Nuclear Energy, 1997, 24, 177-186.	0.9	19
39	Effect of flow rate and power perturbations on dose rates due to coolant activity in low-power research reactors. Annals of Nuclear Energy, 1993, 20, 381-390.	0.9	7
40	Neutron Spectra and Flux Calculation in Thermalizing Regions in Liquid-Metal Reactors. Nuclear Science and Engineering, 1992, 110, 168-176.	0.5	1
41	Study of Coolant Activation and Dose Rates with Flow Rate and Power Perturbations in Pool-Type Research Reactors. Nuclear Technology, 1991, 96, 237-247.	0.7	9