

# Nasir M Mirza

## List of Publications by Year in descending order

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papers

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citations

623574

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752573

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times ranked

222  
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#	ARTICLE	IF	CITATIONS
1	Core loading pattern optimization of a typical two-loop 300MWe PWR using Simulated Annealing (SA), novel crossover Genetic Algorithms (GA) and hybrid GA(SA) schemes. <i>Annals of Nuclear Energy</i> , 2014, 65, 122-131.	0.9	61
2	Bio-inspired heuristics for layer thickness optimization in multilayer piezoelectric transducer for broadband structures. <i>Soft Computing</i> , 2019, 23, 3449-3463.	2.1	36
3	Sensitivity of reactivity insertion limits with respect to safety parameters in a typical MTR. <i>Annals of Nuclear Energy</i> , 1999, 26, 1517-1535.	0.9	31
4	Simulation of reactivity transients in current MTRs. <i>Annals of Nuclear Energy</i> , 1998, 25, 1465-1484.	0.9	30
5	Study of the void coefficients of reactivity in a typical pool type research reactor. <i>Annals of Nuclear Energy</i> , 1997, 24, 177-186.	0.9	19
6	Kinetic simulation of fission product activity in primary coolant of typical PWRs under power perturbations. <i>Nuclear Engineering and Design</i> , 2007, 237, 199-205.	0.8	19
7	Simulation of corrosion product activity in pressurized water reactors under flow rate transients. <i>Annals of Nuclear Energy</i> , 1998, 25, 331-345.	0.9	17
8	Modeling and simulation of corrosion product activity in pressurized water reactors under power perturbations. <i>Annals of Nuclear Energy</i> , 1999, 26, 561-578.	0.9	17
9	Source term evaluation for the upgraded LEU Pakistan Research Reactor-1 under severe accidents. <i>Nuclear Engineering and Design</i> , 2010, 240, 3740-3750.	0.8	15
10	Computer simulation of corrosion product activity in primary coolants of a typical PWR under flow rate transients and linearly accelerating corrosion. <i>Annals of Nuclear Energy</i> , 2003, 30, 831-851.	0.9	14
11	Kinetic study of corrosion product activity in primary coolant pipes of a typical PWR under flow rate transients and linearly increasing corrosion rates. <i>Journal of Nuclear Materials</i> , 2005, 346, 282-292.	1.3	14
12	Simulation of corrosion product activity for nonlinearly rising corrosion on inner surfaces of primary coolant pipes of a typical PWR under flow rate transients. <i>Applied Radiation and Isotopes</i> , 2005, 62, 681-692.	0.7	14
13	Biologically inspired computing framework for solving two-point boundary value problems using differential evolution. <i>Neural Computing and Applications</i> , 2017, 28, 2165-2179.	3.2	14
14	Differential evolution based computation intelligence solver for elliptic partial differential equations. <i>Frontiers of Information Technology and Electronic Engineering</i> , 2019, 20, 1445-1456.	1.5	14
15	Comparative study of actinide and fission product inventory of HEU and potential LEU fuels for MNSRs. <i>Progress in Nuclear Energy</i> , 2009, 51, 129-134.	1.3	13
16	A comparative neutronic study of the standard HEU core and various potential LEU alternatives for a typical MNSR system. <i>Nuclear Engineering and Design</i> , 2008, 238, 2302-2307.	0.8	11
17	Kinetic study of fission product activity released inside containment under loss of coolant transients in a typical MTR system. <i>Applied Radiation and Isotopes</i> , 2012, 70, 2711-2719.	0.7	11
18	Effect of flow rate transients on fission product activity in primary coolant of PWRs. <i>Progress in Nuclear Energy</i> , 2007, 49, 120-129.	1.3	10

#	ARTICLE	IF	CITATIONS
19	Post-shutdown decay power and radionuclide inventories in the discharged fuels of HEU and potential LEU miniature neutron source reactors. <i>Annals of Nuclear Energy</i> , 2010, 37, 701-706.	0.9	10
20	Sensitivity analysis of fission product activity in primary coolant of typical PWRs. <i>Progress in Nuclear Energy</i> , 2011, 53, 245-249.	1.3	10
21	Study of Coolant Activation and Dose Rates with Flow Rate and Power Perturbations in Pool-Type Research Reactors. <i>Nuclear Technology</i> , 1991, 96, 237-247.	0.7	9
22	Stochastic simulation of fission product activity in primary coolant due to fuel rod failures in typical PWRs under power transients. <i>Journal of Nuclear Materials</i> , 2008, 372, 132-140.	1.3	9
23	Time-dependent corrosion product activity in a typical PWR due to changes in coolant chemistry for long-term fuel cycles. <i>Progress in Nuclear Energy</i> , 2012, 58, 100-107.	1.3	9
24	Effect of flow rate and power perturbations on dose rates due to coolant activity in low-power research reactors. <i>Annals of Nuclear Energy</i> , 1993, 20, 381-390.	0.9	7
25	Analysis of core life-time and neutronic parameters for HEU and potential LEU/MEU fuels in a typical MNSR. <i>Annals of Nuclear Energy</i> , 2012, 47, 46-52.	0.9	7
26	Two-group, three-dimensional model based study of reactivity induced transients in upgraded LEU material test reactors. <i>Annals of Nuclear Energy</i> , 2008, 35, 647-655.	0.9	6
27	Static and dynamic sensitivity analysis of corrosion product activity in primary coolant circuits of pressurized water reactors. <i>Progress in Nuclear Energy</i> , 2010, 52, 648-654.	1.3	6
28	Simultaneous multiple reactivity insertions in a typical MTR-type research reactor having U <sub>3</sub> Si <sub>2</sub> -Al fuel. <i>Annals of Nuclear Energy</i> , 2015, 85, 869-878.	0.9	5
29	Effect of Kinetic Parameters on Simultaneous Ramp Reactivity Insertion Plus Beam Tube Flooding Accident in a Typical Low Enriched U <sub>3</sub> Si <sub>2</sub> -Al Fuel-Based Material Testing Reactor-Type Research Reactor. <i>Nuclear Engineering and Technology</i> , 2017, 49, 700-709.	1.1	5
30	Study of successive ramp reactivity insertions in typical pool-type research reactors. <i>Progress in Nuclear Energy</i> , 2013, 66, 115-123.	1.3	4
31	Effect of high density dispersion fuels on transient behavior of MTR type research reactor under multiple reactivity transients. <i>Progress in Nuclear Energy</i> , 2015, 85, 511-517.	1.3	4
32	Simulation of corrosion product activity in extended operating cycles of PWRs under flow rate transient and nonlinearly rising corrosion rates coupled with pH effects. <i>Nuclear Engineering and Design</i> , 2012, 249, 388-399.	0.8	3
33	Comparative study of MIRD, experimental and GEANT4 simulations for uniformly distributed I-131 in cylindrical and spherical thyroid models. <i>Radiation Measurements</i> , 2012, 47, 406-409.	0.7	2
34	Absorbed dose estimations of <sup>131</sup> I for critical organs using the GEANT4 Monte Carlo simulation code. <i>Chinese Physics C</i> , 2012, 36, 1150-1156.	1.5	2
35	Determination of age specific <sup>131</sup> I S-factor values for thyroid using anthropomorphic phantom in geant4 simulations. <i>Applied Radiation and Isotopes</i> , 2014, 90, 15-22.	0.7	2
36	Parametric Study of Time-Dependent Corrosion Product Activity due to <sup>56</sup> Mn, <sup>58</sup> Co, and <sup>60</sup> Co in the Primary Coolant Circuit of a Typical Pressurized Water Reactor. <i>Journal of Chemistry</i> , 2015, 2015, 1-10.	0.9	2

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37	Parametric study of iodine-129 releases from nuclear fuel to fuel-clad gap & primary coolant in PWRs. Annals of Nuclear Energy, 2019, 128, 181-189.	0.9	2
38	Neutron Spectra and Flux Calculation in Thermalizing Regions in Liquid-Metal Reactors. Nuclear Science and Engineering, 1992, 110, 168-176.	0.5	1
39	Experimental study of effect of void volume fraction on neutron diffusion parameters in water. Radiation Physics and Chemistry, 2002, 64, 349-357.	1.4	0
40	Modeling and simulation of release of radioactivity from a typical MTR type research reactor under accidental conditions. , 2009, , .		0
41	Response to: Comment on the paper "Post-Shutdown decay power and radionuclide inventories in the discharged fuels of HEU and potential LEU miniature neutron source reactors" by Mirza, S.M., Khan, A., Mirza N.M. [Ann. Nucl. Energy 37 (2010) 701-706]. Annals of Nuclear Energy, 2011, 38, 2865-2866.	0.9	0