Yoshiaki Oka

List of Publications by Year
in descending order

222	4,482	117625	133252
papers	citations	34 h-index	59 g-index

228	228	228	1286
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Risks and benefits of evacuation in TEPCO's Fukushima Daiichi nuclear power station accident. Progress in Nuclear Energy, 2022, 148, 104222.	2.9	5
2	Sensitivity study of melt behavior of Fukushima Daiichi unit 1 type accident with MELCOR code and MPS method. Journal of Nuclear Science and Technology, 2015, 52, 109-121.	1.3	16
3	Fuel rod behavior under normal operating conditions in Super Fast Reactor with high power density. Nuclear Engineering and Design, 2015, 289, 166-174.	1.7	2
4	Analysis of accidents and abnormal transients of a high breeding fast reactor cooled by supercritical-pressure light water. Nuclear Engineering and Design, 2015, 295, 228-238.	1.7	0
5	CFD analysis of coolant channel geometries for a tightly packed fuel rods assembly at subcritical pressure. Nuclear Engineering and Design, 2015, 284, 115-129.	1.7	3
6	Passive safety system of a super fast reactor. Nuclear Engineering and Design, 2015, 289, 117-125.	1.7	7
7	Subchannel analysis with turbulent mixing rate of supercritical pressure fluid. Nuclear Engineering and Design, 2015, 287, 119-130.	1.7	4
8	CFD analysis of coolant channel geometries for a tightly packed fuel rods assembly of Super FBR. Nuclear Engineering and Design, 2015, 288, 119-129.	1.7	4
9	Single pass core design for a Super Fast Reactor. Annals of Nuclear Energy, 2015, 80, 451-459.	1.8	10
10	3D simulation of eutectic interaction of Pbâ€“Sn system using Moving Particle Semi-implicit (MPS) method. Annals of Nuclear Energy, 2015, 81, 26-33.	1.8	19
11	Analysis of anticipated transient without scram of a Super Fast Reactor with single flow pass core. Annals of Nuclear Energy, 2015, 75, 54-63.	1.8	4
12	Implications and Lessons for Advanced Reactor Design and Operation. , 2015, , 223-258.		0
13	Reconstruction of cell homogenized macroscopic cross sections for analyzing fast and thermal coupled cores using the SRAC system. Journal of Nuclear Science and Technology, 2014, 51, 645-655.	1.3	1
14	Accuracy of nuclear design of fast and thermal neutron coupled core by SRAC. Progress in Nuclear Energy, 2014, 71, 82-88.	2.9	4
15	Experimental and numerical study of stratification and solidification/melting behaviors. Nuclear Engineering and Design, 2014, 272, 109-117.	1.7	38
16	Molten uranium eutectic interaction on iron-alloy by MPS method. Nuclear Engineering and Design, 2014, 278, 387-394.	1.7	23
17	Numerical simulation of the SURC-2 and SURC-4 MCCI experiments by MPS method. Annals of Nuclear Energy, 2014, 73, 46-52.	1.8	31
18	Improvements of two-pass core design for super fast reactor. Annals of Nuclear Energy, 2014, 69, 108-115.	1.8	4

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19	Core design of super LWR with double tube water rods. Nuclear Engineering and Design, 2014, 269, 340-348.	1.7	8
20	Improved single pass core design for high temperature Super LWR. Nuclear Engineering and Design, 2014, 267, 100-108.	1.7	14
21	Numerical investigation on melt freezing behavior in a tube by MPS method. Nuclear Engineering and Design, 2014, 273, 440-448.	1.7	36
22	Numerical investigation of grid spacer effect on heat transfer of supercritical water flows in a tight rod bundle. International Journal of Thermal Sciences, 2014, 76, 245-257.	4.9	46
23	Numerical investigation on practicability of reducing MCST by using grid spacer in a tight rod bundle. Nuclear Engineering and Design, 2014, 270, 198-208.	1.7	10
24	Review of R&D for supercritical water cooled reactors. Progress in Nuclear Energy, 2014, 77, 282-299.	2.9	41
25	Accidents and transients analyses of a super fast reactor with single flow pass core. Nuclear Engineering and Design, 2014, 273, 165-174.	1.7	11
26	Numerical analysis of freezing controlled penetration behavior of the molten core debris in an instrument tube with MPS. Annals of Nuclear Energy, 2014, 71, 322-332.	1.8	26
27	Safety analysis of a Super LWR with double tube water rods. Nuclear Engineering and Design, 2014, 266, 129-136.	1.7	4
28	Analysis of Melt Behavior in a Cold Tube by MPS Method. , 2014, , .		0
29	Numerical Analysis of Crust Behavior of Molten Core and Concrete Interaction by Using MPS Method. , 2014, , .		0
30	Nuclear Reactor Calculations. An Advanced Course in Nuclear Engineering, 2014, , 49-126.	0.1	6
31	Reactor Design and Safety. , 2014, , 21-248.		1
32	Safety analysis of a supercritical water cooled fast reactor with all-upward two-pass flow. Annals of Nuclear Energy, 2013, 59, 1-9.	1.8	13
33	Experiments and MPS analysis of stratification behavior of two immiscible fluids. Nuclear Engineering and Design, 2013, 265, 210-221.	1.7	32
34	Single-pass core design of a low-temperature Super LWR. Journal of Nuclear Science and Technology, 2013, 50, 1129-1138.	1.3	12
35	Core design for super fast reactor with all upward flow core cooling. Annals of Nuclear Energy, 2013, 57, 221-229.	1.8	25
36	Total loss of flow accident characteristics of Super FR with new coolant flow. Nuclear Engineering and Design, 2013, 257, 155-160.	1.7	5

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37	Time dependent start-up thermal analysis of a Super Fast Reactor. Nuclear Engineering and Design, 2013, 263, 129-137.	1.7	6
38	High Breeding Core of a Supercritical-Pressure Light Water Cooled Fast Reactor. , 2013, , .		5
39	Numerical simulation on inertia controlled steam bubble condensation using MPS. , 2013, , .		0
40	Plutonium breeding of light water cooled fast reactors. Journal of Nuclear Science and Technology, 2013, 50, 15-20.	1.3	16
41	S083023 Stratification Behavior of Two Fluids by Gravity : (2) Analysis with MPS Method. The Proceedings of Mechanical Engineering Congress Japan, 2013, 2013, _S083023-1-_S083023-5.	0.0	0
42	S083022 Stratification Behavior of Two Fluids by Gravity : (1) Experimental Observation. The Proceedings of Mechanical Engineering Congress Japan, 2013, 2013, _S083022-1-_S083022-5.	0.0	0
43	Safety Analysis of a Super Fast Reactor With Upward Flow Cooling in Two Pass at Supercritical Pressure. , 2013, , .		0
44	Ex-vessel molten core solidification behavior by moving particle semi-implicit method. Journal of Nuclear Science and Technology, 2012, 49, 1156-1164.	1.3	52
45	Numerical investigation on coalescence of bubble pairs rising in a stagnant liquid. Chemical Engineering Science, 2011, 66, 5055-5063.	3.8	104
46	LOCA Analysis of Super Fast Reactor. Journal of Nuclear Science and Technology, 2011, 48, 1289-1299.	1.3	1
47	Study on the LLFPs transmutation in a super-critical water-cooled fast reactor. Nuclear Engineering and Design, 2011, 241, 395-401.	1.7	10
48	CFD analyses in tight-lattice subchannels and seven-rod bundle geometries of a Super Fast Reactor. Nuclear Engineering and Design, 2011, 241, 1656-1666.	1.7	6
49	Improvements of Feedwater Controller for the Super Fast Reactor. Journal of Nuclear Science and Technology, 2010, 47, 1155-1164.	1.3	14
50	Numerical Solution on Spherical Vacuum Bubble Collapse Using MPS Method. Journal of Engineering for Gas Turbines and Power, 2010, 132, .	1.1	4
51	Numerical computation of thermally controlled steam bubble condensation using Moving Particle Semi-implicit (MPS) method. Annals of Nuclear Energy, 2010, 37, 5-15.	1.8	73
52	Numerical analysis of the onset of droplet entrainment in annular two-phase flow by hybrid method. Annals of Nuclear Energy, 2010, 37, 230-240.	1.8	16
53	Safety analysis of a supercritical-pressure water-cooled fast reactor under supercritical pressure. Nuclear Engineering and Design, 2010, 240, 1218-1228.	1.7	23
54	Analysis of fuel rod behavior under normal operating conditions in Super Fast Reactor. Nuclear Engineering and Design, 2010, 240, 1450-1457.	1.7	1

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55	Numerical investigation on bubble dynamics during flow boiling using moving particle semi-implicit method. Nuclear Engineering and Design, 2010, 240, 3830-3840.	1.7	55
56	CFD analysis of heat transfer in subchannels of a Super Fast Reactor. Nuclear Engineering and Design, 2010, 240, 1819-1829.	1.7	11
57	Super Light Water Reactors and Super Fast Reactors. , 2010, , .		43
58	Numerical Simulation on Direct Contact Condensation of Single Bubble in Subcooled Water using MPS method. AIP Conference Proceedings, 2010, , .	0.4	1
59	Core Design. , 2010, , 79-220.		0
60	Parallel Computation for Particle-Grid Hybrid Method. , 2010, , .		0
61	Plant Startup and Stability. , 2010, , 269-347.		0
62	THREE-DIMENSIONAL CORE DESIGN OF A SUPER FAST REACTOR WITH A HIGH POWER DENSITY. Nuclear Engineering and Technology, 2010, 42, 47-54.	2.3	0
63	Measurements of Neutron Capture Cross Section of²³⁷Np for Fast Neutrons. Journal of Nuclear Science and Technology, 2009, 46, 460-468.	1.3	3
64	Three-dimensional core analysis on a super fast reactor with negative local void reactivity. Nuclear Engineering and Design, 2009, 239, 408-417.	1.7	13
65	Thermal and stability considerations for a supercritical water-cooled fast reactor with downward-flow channels during power-raising phase of plant startup. Nuclear Engineering and Design, 2009, 239, 665-679.	1.7	25
66	Numerical simulation on void bubble dynamics using moving particle semi-implicit method. Nuclear Engineering and Design, 2009, 239, 2382-2390.	1.7	38
67	The Study of Hybrid Method in the Onset of Droplet Escaping From the Film. , 2009, , .		0
68	Numerical Solution on Spherical Vacuum Bubble Collapse Using MPS Method. , 2009, , .		0
69	AESJ Members are Encouraged to Get PE Qualification. Atomos, 2009, 51, 407-409.	0.0	0
70	Fuel, Core Design and Subchannel Analysis of a Superfast Reactor. Journal of Nuclear Science and Technology, 2008, 45, 138-148.	1.3	46
71	Fuel, Core Design and Subchannel Analysis of a Superfast Reactor. Journal of Nuclear Science and Technology, 2008, 45, 138-148.	1.3	20
72	Global COE Program â€œNuclear Education and Research Initiativeâ€• Atomos, 2008, 50, 92-96.	0.0	0

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73	Simulation of a Single Bubble Rising with Hybrid Particle-Mesh Method. Journal of Nuclear Science and Technology, 2007, 44, 886-893.	1.3	17
74	ATWS Characteristics of Super LWR with/without Alternative Action. Journal of Nuclear Science and Technology, 2007, 44, 572-589.	1.3	11
75	Hamiltonian moving-particle semi-implicit (HMPS) method for incompressible fluid flows. Computer Methods in Applied Mechanics and Engineering, 2007, 196, 2876-2894.	6.6	60
76	Numerical investigation of heat transfer in upward flows of supercritical water in circular tubes and tight fuel rod bundles. Nuclear Engineering and Design, 2007, 237, 420-430.	1.7	153
77	Subchannel analysis of supercritical light water-cooled fast reactor assembly. Nuclear Engineering and Design, 2007, 237, 1096-1105.	1.7	22
78	ATWS Characteristics of Super LWR with/without Alternative Action. Journal of Nuclear Science and Technology, 2007, 44, 572-589.	1.3	3
79	Simulation of a Single Bubble Rising with Hybrid Particle-Mesh Method. Journal of Nuclear Science and Technology, 2007, 44, 886-893.	1.3	2
80	SAFETY OF THE SUPER LWR. Nuclear Engineering and Technology, 2007, 39, 257-272.	2.3	27
81	Numerical Analyses of Flashing Jet Structure and Droplet Size Characteristics. Journal of Nuclear Science and Technology, 2006, 43, 285-294.	1.3	6
82	Development of Statistical Thermal Design Procedure to Evaluate Engineering Uncertainty of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 32-42.	1.3	12
83	Conceptual design of compact supercritical water-cooled fast reactor with thermal hydraulic coupling. Annals of Nuclear Energy, 2006, 33, 945-956.	1.8	57
84	Direct simulation of flashing liquid jets using the MPS method. International Journal of Heat and Mass Transfer, 2006, 49, 402-405.	4.8	26
85	Thermo-mechanical analysis of supercritical pressure light water-cooled fast reactor fuel rod by FEMAXI-6 code. Annals of Nuclear Energy, 2006, 33, 1379-1390.	1.8	9
86	Fuel and Core Design of Super Light Water Reactor with Low Leakage Fuel Loading Pattern. Journal of Nuclear Science and Technology, 2006, 43, 129-139.	1.3	41
87	LOCA Analysis of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 231-241.	1.3	15
88	Fuel and Core Design of Super Light Water Reactor with Low Leakage Fuel Loading Pattern. Journal of Nuclear Science and Technology, 2006, 43, 129-139.	1.3	13
89	LOCA Analysis of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 231-241.	1.3	3
90	Numerical Analyses of Flashing Jet Structure and Droplet Size Characteristics. Journal of Nuclear Science and Technology, 2006, 43, 285-294.	1.3	3

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91	Development of Statistical Thermal Design Procedure to Evaluate Engineering Uncertainty of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 32-42.	1.3	4
92	Dynamic Analysis of Elastic Solids by MPS Method. Nihon Kikai Gakkai Ronbunshu, A Hen/Transactions of the Japan Society of Mechanical Engineers, Part A, 2005, 71, 16-22.	0.2	22
93	A hybrid particle-mesh method for viscous, incompressible, multiphase flows. Journal of Computational Physics, 2005, 202, 65-93.	3.8	119
94	Simulation of drop deposition process in annular mist flow using three-dimensional particle method. Nuclear Engineering and Design, 2005, 235, 1687-1697.	1.7	16
95	Safety of Super LWR, (I) Safety System Design. Journal of Nuclear Science and Technology, 2005, 42, 927-934.	1.3	32
96	Three-dimensional Core Design of High Temperature Supercritical-Pressure Light Water Reactor with Neutronic and Thermal-Hydraulic Coupling. Journal of Nuclear Science and Technology, 2005, 42, 8-19.	1.3	60
97	Safety of Super LWR, (II). Journal of Nuclear Science and Technology, 2005, 42, 935-948.	1.3	38
98	Thermal and Stability Considerations of Super LWR during Sliding Pressure Startup. Journal of Nuclear Science and Technology, 2005, 42, 537-548.	1.3	23
99	Evaluation of the Energy Conversion Ratio of Vapor Explosions for the Assessment of Nuclear Reactor Safety. Journal of Nuclear Science and Technology, 2005, 42, 28-39.	1.3	3
100	Thermal and Stability Considerations of Super LWR during Sliding Pressure Startup. Journal of Nuclear Science and Technology, 2005, 42, 537-548.	1.3	1
101	Safety of Super LWR, (I) Safety System Design. Journal of Nuclear Science and Technology, 2005, 42, 927-934.	1.3	6
102	Safety of Super LWR, (II) Safety Analysis at Supercritical Pressure. Journal of Nuclear Science and Technology, 2005, 42, 935-948.	1.3	10
103	A Linear Stability Analysis of Supercritical Water Reactors, (II). Journal of Nuclear Science and Technology, 2004, 41, 1176-1186.	1.3	36
104	Modelling of a single drop impact onto liquid film using particle method. International Journal for Numerical Methods in Fluids, 2004, 45, 1009-1023.	1.6	43
105	A Linear Stability Analysis of Supercritical Water Reactors, (I). Journal of Nuclear Science and Technology, 2004, 41, 1166-1175.	1.3	63
106	Startup Thermal Analysis of a High-Temperature Supercritical-Pressure Light Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 790-801.	1.3	19
107	Numerical Simulation of Liquid Drop Deposition in Annular-Mist Flow Regime of Boiling Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 569-578.	1.3	32
108	Numerical Analysis of Jet Breakup Behavior Using Particle Method. Journal of Nuclear Science and Technology, 2004, 41, 715-722.	1.3	49

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109	The Role of Fragmentation Mechanism in Large-Scale Vapor Explosions. JSME International Journal Series B, 2004, 47, 268-276.	0.3	0
110	A Linear Stability Analysis of Supercritical Water Reactors, (I) Thermal-Hydraulic Stability. Journal of Nuclear Science and Technology, 2004, 41, 1166-1175.	1.3	3
111	A Linear Stability Analysis of Supercritical Water Reactors, (II) Coupled Neutronic Thermal-Hydraulic Stability. Journal of Nuclear Science and Technology, 2004, 41, 1176-1186.	1.3	2
112	Numerical Simulation of Liquid Drop Deposition in Annular-Mist Flow Regime of Boiling Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 569-578.	1.3	7
113	Numerical Analysis of Jet Breakup Behavior Using Particle Method. Journal of Nuclear Science and Technology, 2004, 41, 715-722.	1.3	8
114	Startup Thermal Analysis of a High-Temperature Supercritical-Pressure Light Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 790-801.	1.3	5
115	Three-dimensional numerical simulation of droplet deposition. The Proceedings of the JSME Annual Meeting, 2004, 2004.3, 207-208.	0.0	0
116	Two-dimensional simulation of drop deformation and breakup at around the critical Weber number. Nuclear Engineering and Design, 2003, 225, 37-48.	1.7	63
117	Numerical and Theoretical Investigation of Effect of Density Ratio on the Critical Weber Number of Droplet Breakup. Journal of Nuclear Science and Technology, 2003, 40, 501-508.	1.3	32
118	Control of a High Temperature Supercritical Pressure Light Water Cooled and Moderated Reactor with Water Rods. Journal of Nuclear Science and Technology, 2003, 40, 298-306.	1.3	42
119	Numerical Analysis of Pressure Propagation and Energy Conversion Ratio in Sodium Vapor Explosions. Nuclear Technology, 2003, 144, 324-336.	1.2	0
120	Control of a High Temperature Supercritical Pressure Light Water Cooled and Moderated Reactor with Water Rods. Journal of Nuclear Science and Technology, 2003, 40, 298-306.	1.3	14
121	Numerical and Theoretical Investigation of Effect of Density Ratio on the Critical Weber Number of Droplet Breakup. Journal of Nuclear Science and Technology, 2003, 40, 501-508.	1.3	8
122	ICONE11-36047 The Role of Fragmentation Mechanism in Large-scale Vapor Explosions. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2003, 2003, 221.	0.0	1
123	Investigation on Energetics of Ex-vessel Vapor Explosion Based on Spontaneous Nucleation Fragmentation. Journal of Nuclear Science and Technology, 2002, 39, 31-39.	1.3	2
124	Light Water Cooled, High Temperature and High Performance Nuclear Power Plants Concept of Once-through Coolant Cycle, Supercritical-pressure, Light Water Cooled Nuclear Reactors. Nippon Genshiryoku Gakkaishi/Journal of the Atomic Energy Society of Japan, 2002, 44, 600-605.	0.0	2
125	Relationship between the structure of vapor explosion and fragmentation mechanisms. Nuclear Engineering and Design, 2002, 216, 121-137.	1.7	5
126	Numerical Analysis of Heat Transfer Deterioration. The Reference Collection of Annual Meeting, 2002, VIII.02.1, 169-170.	0.0	0

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127	Coupled Analysis of Fluid and Structure Using Particle Method. The Reference Collection of Annual Meeting, 2002, VIII.02.1, 348-349.	0.0	0
128	Conceptual Design of Once-through, Supercritical-pressure Light Water Cooled Nuclear Power Plants. The Reference Collection of Annual Meeting, 2002, VIII.02.1, 163-164.	0.0	1
129	Numerical Analysis of Droplet Breakup Behavior using Particle Method. Journal of Nuclear Science and Technology, 2001, 38, 1057-1064.	1.3	140
130	Numerical Analysis of Jet Injection Behavior for Fuel-Coolant Interaction using Particle Method. Journal of Nuclear Science and Technology, 2001, 38, 174-182.	1.3	84
131	Breeding Ratio Analysis of a Fast Reactor Cooled by Supercritical Light Water. Journal of Nuclear Science and Technology, 2001, 38, 703-710.	1.3	20
132	Refinement of Transient Criteria and Safety Analysis for a High-Temperature Reactor Cooled by Supercritical Water. Nuclear Technology, 2001, 135, 252-264.	1.2	42
133	Startup Thermal Considerations for Supercritical-Pressure Light Water-Cooled Reactors. Nuclear Technology, 2001, 134, 221-230.	1.2	17
134	Direct calculation of bubble growth, departure, and rise in nucleate pool boiling. International Journal of Multiphase Flow, 2001, 27, 277-298.	3.4	144
135	Supercritical-pressure, Once-through Cycle Light Water Cooled Reactor Concept. Journal of Nuclear Science and Technology, 2001, 38, 1081-1089.	1.3	79
136	Numerical Analysis of Droplet Breakup Behavior using Particle Method.. Journal of Nuclear Science and Technology, 2001, 38, 1057-1064.	1.3	21
137	Supercritical-pressure, Once-through Cycle Light Water Cooled Reactor Concept.. Journal of Nuclear Science and Technology, 2001, 38, 1081-1089.	1.3	8
138	Numerical Analysis of Jet Injection Behavior for Fuel-Coolant Interaction using Particle Method.. Journal of Nuclear Science and Technology, 2001, 38, 174-182.	1.3	13
139	Breeding Ratio Analysis of a Fast Reactor Cooled by Supercritical Light Water.. Journal of Nuclear Science and Technology, 2001, 38, 703-710.	1.3	5
140	Subchannel analysis of a fast reactor cooled by supercritical light water. Progress in Nuclear Energy, 2000, 37, 197-204.	2.9	6
141	Radiation Shielding for Fission Reactors. Journal of Nuclear Science and Technology, 2000, 37, 1-10.	1.3	6
142	Numerical analysis of fragmentation mechanisms in vapor explosions. Nuclear Engineering and Design, 1999, 189, 423-433.	1.7	122
143	Core design of a high-temperature fast reactor cooled by supercritical light water. Annals of Nuclear Energy, 1999, 26, 1423-1436.	1.8	29
144	A particle-gridless hybrid method for incompressible flows. International Journal for Numerical Methods in Fluids, 1999, 30, 407-424.	1.6	63

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145	A Mesh-Free Numerical Method for Direct Simulation of Gas-Liquid Phase Interface. Nuclear Science and Engineering, 1999, 133, 192-200.	1.1	28
146	Numerical Analysis of Sloshing with Large Deformation of Elastic Walls and Free Surfaces Using MPS method.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1999, 65, 2954-2960.	0.2	7
147	Numerical analysis of breaking waves using the moving particle semi-implicit method. International Journal for Numerical Methods in Fluids, 1998, 26, 751-769.	1.6	704
148	Measurement of decay heat of fast neutron fission products. Progress in Nuclear Energy, 1998, 32, 53-60.	2.9	1
149	Study of epithermal neutron columns for boron neutron capture therapy. Progress in Nuclear Energy, 1998, 32, 61-70.	2.9	6
150	Development of a LOCA analysis code for the supercritical-pressure light water cooled reactors. Annals of Nuclear Energy, 1998, 25, 1341-1361.	1.8	27
151	Control of a Fast Reactor Cooled by Supercritical Light Water. Nuclear Technology, 1998, 121, 81-92.	1.2	28
152	Pressure- and Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Supercritical-Pressure, Light-Water-Cooled Fast Reactor. Nuclear Technology, 1998, 123, 233-244.	1.2	13
153	Numerical Analysis of Fragmentation Processes of Liquid Metal in Vapor Explosions Using Moving Particle Semi-implicit Method.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1998, 64, 2431-2437.	0.2	7
154	Numerical analysis of breaking waves using the moving particle semiâ€implicit method. International Journal for Numerical Methods in Fluids, 1998, 26, 751-769.	1.6	17
155	Safety analysis of a supercritical pressure, light water cooled and moderated reactor with double tube water rods. Annals of Nuclear Energy, 1997, 24, 1447-1456.	1.8	7
156	Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Light-Water-Cooled, Fast Breeder Reactor Operating at Supercritical Pressure. Journal of Nuclear Science and Technology, 1996, 33, 307-315.	1.3	20
157	UO₂Core Design of a Direct-Cycle Fast Converter Reactor Cooled by Supercritical Water. Nuclear Technology, 1996, 114, 273-284.	1.2	3
158	Negative coolant void reactivity in large fast breeder reactors with hydrogenous moderator layer. Annals of Nuclear Energy, 1996, 23, 1105-1115.	1.8	38
159	Core Design of a Direct-Cycle, Supercritical-Pressure, Light Water Reactor with Double Tube Water Rods. Journal of Nuclear Science and Technology, 1996, 33, 365-373.	1.3	19
160	Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Light-Water-Cooled, Fast Breeder Reactor Operating at Supercritical Pressure.. Journal of Nuclear Science and Technology, 1996, 33, 307-315.	1.3	7
161	Core Design of a Direct-Cycle, Supercritical-Pressure, Light Water Reactor with Double Tube Water Rods.. Journal of Nuclear Science and Technology, 1996, 33, 365-373.	1.3	7
162	Systems Design of Direct-Cycle Supercritical-Water-Cooled Fast Reactors. Nuclear Technology, 1995, 109, 1-10.	1.2	34

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163	Design of water rod cores of a direct cycle supercritical-pressure light water reactor. Annals of Nuclear Energy, 1994, 21, 601-611.	1.8	34
164	Numerical Analysis of Deterioration in Heat Transfer to Supercritical Water.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1994, 60, 2497-2503.	0.2	4
165	A Direct-Cycle, Supercritical- Water-Cooled Fast Breeder Reactor. Journal of Nuclear Science and Technology, 1994, 31, 83-85.	1.3	9
166	Negative Void Reactivity in a Large Liquid-Metal Fast Breeder Reactor with Hydrogenous Moderator (ZrH_{1.7}) Layers. Nuclear Technology, 1994, 107, 15-22.	1.2	5
167	Core Design of a Direct-Cycle, Supercritical-Water-Cooled Fast Breeder Reactor. Nuclear Technology, 1994, 108, 24-32.	1.2	18
168	A Direct-Cycle, Supercritical-Water-Cooled Fast Breeder Reactor.. Journal of Nuclear Science and Technology, 1994, 31, 83-85.	1.3	1
169	Effect of Zirconium-Hydride Layers on Reducing Coolant Void Reactivity of Steam Cooled Fast Breeder Reactors. Journal of Nuclear Science and Technology, 1993, 30, 497-504.	1.3	17
170	Concept and Design of a Supercritical-Pressure, Direct-Cycle Light Water Reactor. Nuclear Technology, 1993, 103, 295-302.	1.2	61
171	International Conference on Design and Safety of Advanced Nuclear Power Plants (ANP'92). Journal of Nuclear Science and Technology, 1993, 30, 358-362.	1.3	0
172	Effect of Zirconium-Hydride Layers on Reducing Coolant Void Reactivity of Steam Cooled Fast Breeder Reactors.. Journal of Nuclear Science and Technology, 1993, 30, 497-504.	1.3	7
173	Direct Cycle Light Water Reactor Operating at Supercritical Pressure. Journal of Nuclear Science and Technology, 1992, 29, 585-588.	1.3	35
174	Direct Cycle Light Water Reactor Operating at Supercritical Pressure.. Journal of Nuclear Science and Technology, 1992, 29, 585-588.	1.3	13
175	Effect of Step Geometry on Reattachment Length in Backward-Facing Step Flow. 1st Report: Experiment.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1991, 57, 2192-2196.	0.2	0
176	Neutronic Feasibility of Supercritical Steam Cooled Fast Breeder Reactor. Journal of Nuclear Science and Technology, 1991, 28, 585-587.	1.3	4
177	Neutronic Feasibility of Supercritical Steam Cooled Fast Breeder Reactor.. Journal of Nuclear Science and Technology, 1991, 28, 585-587.	1.3	6
178	Boundary-fitted coordinate method for incompressible flow using Riemann geometry.. Nippon Genshiryoku Gakkaishi/Journal of the Atomic Energy Society of Japan, 1990, 32, 819-833.	0.0	3
179	Interpolating matrix method: A finite difference method for arbitrary arrangement of mesh points. Journal of Computational Physics, 1988, 75, 444-468.	3.8	14
180	Neutronic and thermodynamic feasibility of 233UF6-237NpF6 fueled transmutation reactor. Nuclear Engineering and Design, 1988, 110, 99-105.	1.7	0

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181	Fusion nuclear systems design and analysis. Fusion Engineering and Design, 1988, 7, 369-376.	1.9	3
182	Development of expert system on personal computer for diagnosis of nuclear reactor malfunctions.. Nippon Genshiryoku Gakkaishi/Journal of the Atomic Energy Society of Japan, 1988, 30, 42-48.	0.0	0
183	Development of Radiation Transport Code in Three-Dimensional (X, Y, Z)Geometry for Shielding Analyses by Direct Integration Method. Journal of Nuclear Science and Technology, 1987, 24, 181-193.	1.3	5
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