

# Yoshiaki Oka

## List of Publications by Year in descending order

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222  
papers

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228  
docs citations

228  
times ranked

1286  
citing authors

| #  | ARTICLE   | IF  | CITATIONS |
|----|---|-----|-----------|
| 1  | Risks and benefits of evacuation in TEPCO's Fukushima Daiichi nuclear power station accident. Progress in Nuclear Energy, 2022, 148, 104222.  | 1.3 | 5         |
| 2  | Sensitivity study of melt behavior of Fukushima Daiichi unit 1 type accident with MELCOR code and MPS method. Journal of Nuclear Science and Technology, 2015, 52, 109-121.                     | 0.7 | 16        |
| 3  | Fuel rod behavior under normal operating conditions in Super Fast Reactor with high power density. Nuclear Engineering and Design, 2015, 289, 166-174.  | 0.8 | 2         |
| 4  | Analysis of accidents and abnormal transients of a high breeding fast reactor cooled by supercritical-pressure light water. Nuclear Engineering and Design, 2015, 295, 228-238.                 | 0.8 | 0         |
| 5  | CFD analysis of coolant channel geometries for a tightly packed fuel rods assembly at subcritical pressure. Nuclear Engineering and Design, 2015, 284, 115-129.                                 | 0.8 | 3         |
| 6  | Passive safety system of a super fast reactor. Nuclear Engineering and Design, 2015, 289, 117-125.  | 0.8 | 7         |
| 7  | Subchannel analysis with turbulent mixing rate of supercritical pressure fluid. Nuclear Engineering and Design, 2015, 287, 119-130.   | 0.8 | 4         |
| 8  | CFD analysis of coolant channel geometries for a tightly packed fuel rods assembly of Super FBR. Nuclear Engineering and Design, 2015, 288, 119-129.  | 0.8 | 4         |
| 9  | Single pass core design for a Super Fast Reactor. Annals of Nuclear Energy, 2015, 80, 451-459.  | 0.9 | 10        |
| 10 | 3D simulation of eutectic interaction of Pb-Sn system using Moving Particle Semi-implicit (MPS) method. Annals of Nuclear Energy, 2015, 81, 26-33.  | 0.9 | 19        |
| 11 | Analysis of anticipated transient without scram of a Super Fast Reactor with single flow pass core. Annals of Nuclear Energy, 2015, 75, 54-63.  | 0.9 | 4         |
| 12 | Implications and Lessons for Advanced Reactor Design and Operation. , 2015, , 223-258.  |     | 0         |
| 13 | Reconstruction of cell homogenized macroscopic cross sections for analyzing fast and thermal coupled cores using the SRAC system. Journal of Nuclear Science and Technology, 2014, 51, 645-655. | 0.7 | 1         |
| 14 | Accuracy of nuclear design of fast and thermal neutron coupled core by SRAC. Progress in Nuclear Energy, 2014, 71, 82-88.   | 1.3 | 4         |
| 15 | Experimental and numerical study of stratification and solidification/melting behaviors. Nuclear Engineering and Design, 2014, 272, 109-117.  | 0.8 | 38        |
| 16 | Molten uranium eutectic interaction on iron-alloy by MPS method. Nuclear Engineering and Design, 2014, 278, 387-394.  | 0.8 | 23        |
| 17 | Numerical simulation of the SURC-2 and SURC-4 MCCI experiments by MPS method. Annals of Nuclear Energy, 2014, 73, 46-52.  | 0.9 | 31        |
| 18 | Improvements of two-pass core design for super fast reactor. Annals of Nuclear Energy, 2014, 69, 108-115.   | 0.9 | 4         |

| #  | ARTICLE  | IF  | CITATIONS |
|----|--|-----|-----------|
| 19 | Core design of super LWR with double tube water rods. Nuclear Engineering and Design, 2014, 269, 340-348.  | 0.8 | 8         |
| 20 | Improved single pass core design for high temperature Super LWR. Nuclear Engineering and Design, 2014, 267, 100-108.   | 0.8 | 14        |
| 21 | Numerical investigation on melt freezing behavior in a tube by MPS method. Nuclear Engineering and Design, 2014, 273, 440-448.   | 0.8 | 36        |
| 22 | Numerical investigation of grid spacer effect on heat transfer of supercritical water flows in a tight rod bundle. International Journal of Thermal Sciences, 2014, 76, 245-257. | 2.6 | 46        |
| 23 | Numerical investigation on practicability of reducing MCST by using grid spacer in a tight rod bundle. Nuclear Engineering and Design, 2014, 270, 198-208.                       | 0.8 | 10        |
| 24 | Review of R&D for supercritical water cooled reactors. Progress in Nuclear Energy, 2014, 77, 282-299.  | 1.3 | 41        |
| 25 | Accidents and transients analyses of a super fast reactor with single flow pass core. Nuclear Engineering and Design, 2014, 273, 165-174.  | 0.8 | 11        |
| 26 | Numerical analysis of freezing controlled penetration behavior of the molten core debris in an instrument tube with MPS. Annals of Nuclear Energy, 2014, 71, 322-332.            | 0.9 | 26        |
| 27 | Safety analysis of a Super LWR with double tube water rods. Nuclear Engineering and Design, 2014, 266, 129-136.  | 0.8 | 4         |
| 28 | Analysis of Melt Behavior in a Cold Tube by MPS Method. , 2014, , .  |     | 0         |
| 29 | Numerical Analysis of Crust Behavior of Molten Core and Concrete Interaction by Using MPS Method. , 2014, , .  |     | 0         |
| 30 | Nuclear Reactor Calculations. An Advanced Course in Nuclear Engineering, 2014, , 49-126.   | 0.1 | 6         |
| 31 | Reactor Design and Safety. , 2014, , 21-248.   |     | 1         |
| 32 | Safety analysis of a supercritical water cooled fast reactor with all-upward two-pass flow. Annals of Nuclear Energy, 2013, 59, 1-9.   | 0.9 | 13        |
| 33 | Experiments and MPS analysis of stratification behavior of two immiscible fluids. Nuclear Engineering and Design, 2013, 265, 210-221.  | 0.8 | 32        |
| 34 | Single-pass core design of a low-temperature Super LWR. Journal of Nuclear Science and Technology, 2013, 50, 1129-1138.  | 0.7 | 12        |
| 35 | Core design for super fast reactor with all upward flow core cooling. Annals of Nuclear Energy, 2013, 57, 221-229.   | 0.9 | 25        |
| 36 | Total loss of flow accident characteristics of Super FR with new coolant flow. Nuclear Engineering and Design, 2013, 257, 155-160.   | 0.8 | 5         |

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|----|---|-----|-----------|
| 37 | Time dependent start-up thermal analysis of a Super Fast Reactor. Nuclear Engineering and Design, 2013, 263, 129-137.   | 0.8 | 6         |
| 38 | High Breeding Core of a Supercritical-Pressure Light Water Cooled Fast Reactor. , 2013, , .   |     | 5         |
| 39 | Numerical simulation on inertia controlled steam bubble condensation using MPS. , 2013, , .   |     | 0         |
| 40 | Plutonium breeding of light water cooled fast reactors. Journal of Nuclear Science and Technology, 2013, 50, 15-20.   | 0.7 | 16        |
| 41 | S083023 Stratification Behavior of Two Fluids by Gravity : (2) Analysis with MPS Method. The Proceedings of Mechanical Engineering Congress Japan, 2013, 2013, _S083023-1-_S083023-5. | 0.0 | 0         |
| 42 | S083022 Stratification Behavior of Two Fluids by Gravity : (1) Experimental Observation. The Proceedings of Mechanical Engineering Congress Japan, 2013, 2013, _S083022-1-_S083022-5. | 0.0 | 0         |
| 43 | Safety Analysis of a Super Fast Reactor With Upward Flow Cooling in Two Pass at Supercritical Pressure. , 2013, , .   |     | 0         |
| 44 | Ex-vessel molten core solidification behavior by moving particle semi-implicit method. Journal of Nuclear Science and Technology, 2012, 49, 1156-1164.                                | 0.7 | 52        |
| 45 | Numerical investigation on coalescence of bubble pairs rising in a stagnant liquid. Chemical Engineering Science, 2011, 66, 5055-5063.  | 1.9 | 104       |
| 46 | LOCA Analysis of Super Fast Reactor. Journal of Nuclear Science and Technology, 2011, 48, 1289-1299.  | 0.7 | 1         |
| 47 | Study on the LLFPs transmutation in a super-critical water-cooled fast reactor. Nuclear Engineering and Design, 2011, 241, 395-401.   | 0.8 | 10        |
| 48 | CFD analyses in tight-lattice subchannels and seven-rod bundle geometries of a Super Fast Reactor. Nuclear Engineering and Design, 2011, 241, 1656-1666.                              | 0.8 | 6         |
| 49 | Improvements of Feedwater Controller for the Super Fast Reactor. Journal of Nuclear Science and Technology, 2010, 47, 1155-1164.  | 0.7 | 14        |
| 50 | Numerical Solution on Spherical Vacuum Bubble Collapse Using MPS Method. Journal of Engineering for Gas Turbines and Power, 2010, 132, .  | 0.5 | 4         |
| 51 | Numerical computation of thermally controlled steam bubble condensation using Moving Particle Semi-implicit (MPS) method. Annals of Nuclear Energy, 2010, 37, 5-15.                   | 0.9 | 73        |
| 52 | Numerical analysis of the onset of droplet entrainment in annular two-phase flow by hybrid method. Annals of Nuclear Energy, 2010, 37, 230-240.                                       | 0.9 | 16        |
| 53 | Safety analysis of a supercritical-pressure water-cooled fast reactor under supercritical pressure. Nuclear Engineering and Design, 2010, 240, 1218-1228.                             | 0.8 | 23        |
| 54 | Analysis of fuel rod behavior under normal operating conditions in Super Fast Reactor. Nuclear Engineering and Design, 2010, 240, 1450-1457.  | 0.8 | 1         |

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|----|---|-----|-----------|
| 55 | Numerical investigation on bubble dynamics during flow boiling using moving particle semi-implicit method. Nuclear Engineering and Design, 2010, 240, 3830-3840.  | 0.8 | 55        |
| 56 | CFD analysis of heat transfer in subchannels of a Super Fast Reactor. Nuclear Engineering and Design, 2010, 240, 1819-1829.   | 0.8 | 11        |
| 57 | Super Light Water Reactors and Super Fast Reactors. , 2010, , .   |     | 43        |
| 58 | Numerical Simulation on Direct Contact Condensation of Single Bubble in Subcooled Water using MPS method. AIP Conference Proceedings, 2010, , .   | 0.3 | 1         |
| 59 | Core Design. , 2010, , 79-220.  |     | 0         |
| 60 | Parallel Computation for Particle-Grid Hybrid Method. , 2010, , .   |     | 0         |
| 61 | Plant Startup and Stability. , 2010, , 269-347.   |     | 0         |
| 62 | THREE-DIMENSIONAL CORE DESIGN OF A SUPER FAST REACTOR WITH A HIGH POWER DENSITY. Nuclear Engineering and Technology, 2010, 42, 47-54.   | 1.1 | 0         |
| 63 | Measurements of Neutron Capture Cross Section of <sup>237</sup> Np for Fast Neutrons. Journal of Nuclear Science and Technology, 2009, 46, 460-468.   | 0.7 | 3         |
| 64 | Three-dimensional core analysis on a super fast reactor with negative local void reactivity. Nuclear Engineering and Design, 2009, 239, 408-417.  | 0.8 | 13        |
| 65 | Thermal and stability considerations for a supercritical water-cooled fast reactor with downward-flow channels during power-raising phase of plant startup. Nuclear Engineering and Design, 2009, 239, 665-679. | 0.8 | 25        |
| 66 | Numerical simulation on void bubble dynamics using moving particle semi-implicit method. Nuclear Engineering and Design, 2009, 239, 2382-2390.  | 0.8 | 38        |
| 67 | The Study of Hybrid Method in the Onset of Droplet Escaping From the Film. , 2009, , .  |     | 0         |
| 68 | Numerical Solution on Spherical Vacuum Bubble Collapse Using MPS Method. , 2009, , .  |     | 0         |
| 69 | AESJ Members are Encouraged to Get PE Qualification. Atomos, 2009, 51, 407-409.   | 0.0 | 0         |
| 70 | Fuel, Core Design and Subchannel Analysis of a Superfast Reactor. Journal of Nuclear Science and Technology, 2008, 45, 138-148.   | 0.7 | 46        |
| 71 | Fuel, Core Design and Subchannel Analysis of a Superfast Reactor. Journal of Nuclear Science and Technology, 2008, 45, 138-148.   | 0.7 | 20        |
| 72 | Global COE Program "Nuclear Education and Research Initiative" Atomos, 2008, 50, 92-96.   | 0.0 | 0         |

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|----|---|-----|-----------|
| 73 | Simulation of a Single Bubble Rising with Hybrid Particle-Mesh Method. Journal of Nuclear Science and Technology, 2007, 44, 886-893.  | 0.7 | 17        |
| 74 | ATWS Characteristics of Super LWR with/without Alternative Action. Journal of Nuclear Science and Technology, 2007, 44, 572-589.  | 0.7 | 11        |
| 75 | Hamiltonian moving-particle semi-implicit (HMPS) method for incompressible fluid flows. Computer Methods in Applied Mechanics and Engineering, 2007, 196, 2876-2894.              | 3.4 | 60        |
| 76 | Numerical investigation of heat transfer in upward flows of supercritical water in circular tubes and tight fuel rod bundles. Nuclear Engineering and Design, 2007, 237, 420-430. | 0.8 | 153       |
| 77 | Subchannel analysis of supercritical light water-cooled fast reactor assembly. Nuclear Engineering and Design, 2007, 237, 1096-1105.  | 0.8 | 22        |
| 78 | ATWS Characteristics of Super LWR with/without Alternative Action. Journal of Nuclear Science and Technology, 2007, 44, 572-589.  | 0.7 | 3         |
| 79 | Simulation of a Single Bubble Rising with Hybrid Particle-Mesh Method. Journal of Nuclear Science and Technology, 2007, 44, 886-893.  | 0.7 | 2         |
| 80 | SAFETY OF THE SUPER LWR. Nuclear Engineering and Technology, 2007, 39, 257-272.   | 1.1 | 27        |
| 81 | Numerical Analyses of Flashing Jet Structure and Droplet Size Characteristics. Journal of Nuclear Science and Technology, 2006, 43, 285-294.                                      | 0.7 | 6         |
| 82 | Development of Statistical Thermal Design Procedure to Evaluate Engineering Uncertainty of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 32-42.                 | 0.7 | 12        |
| 83 | Conceptual design of compact supercritical water-cooled fast reactor with thermal hydraulic coupling. Annals of Nuclear Energy, 2006, 33, 945-956.                                | 0.9 | 57        |
| 84 | Direct simulation of flashing liquid jets using the MPS method. International Journal of Heat and Mass Transfer, 2006, 49, 402-405.   | 2.5 | 26        |
| 85 | Thermo-mechanical analysis of supercritical pressure light water-cooled fast reactor fuel rod by FEMAXI-6 code. Annals of Nuclear Energy, 2006, 33, 1379-1390.                    | 0.9 | 9         |
| 86 | Fuel and Core Design of Super Light Water Reactor with Low Leakage Fuel Loading Pattern. Journal of Nuclear Science and Technology, 2006, 43, 129-139.                            | 0.7 | 41        |
| 87 | LOCA Analysis of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 231-241.   | 0.7 | 15        |
| 88 | LOCA Analysis of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 231-241.   | 0.7 | 3         |
| 89 | Numerical Analyses of Flashing Jet Structure and Droplet Size Characteristics. Journal of Nuclear Science and Technology, 2006, 43, 285-294.                                      | 0.7 | 3         |
| 90 | Development of Statistical Thermal Design Procedure to Evaluate Engineering Uncertainty of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 32-42.                 | 0.7 | 4         |

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|-----|--|-----|-----------|
| 91  | Dynamic Analysis of Elastic Solids by MPS Method. Nihon Kikai Gakkai Ronbunshu, A Hen/Transactions of the Japan Society of Mechanical Engineers, Part A, 2005, 71, 16-22.                              | 0.2 | 22        |
| 92  | A hybrid particle-mesh method for viscous, incompressible, multiphase flows. Journal of Computational Physics, 2005, 202, 65-93.   | 1.9 | 119       |
| 93  | Simulation of drop deposition process in annular mist flow using three-dimensional particle method. Nuclear Engineering and Design, 2005, 235, 1687-1697.  | 0.8 | 16        |
| 94  | Safety of Super LWR, (I) Safety System Design. Journal of Nuclear Science and Technology, 2005, 42, 927-934.   | 0.7 | 32        |
| 95  | Three-dimensional Core Design of High Temperature Supercritical-Pressure Light Water Reactor with Neutronic and Thermal-Hydraulic Coupling. Journal of Nuclear Science and Technology, 2005, 42, 8-19. | 0.7 | 60        |
| 96  | Safety of Super LWR, (II). Journal of Nuclear Science and Technology, 2005, 42, 935-948.   | 0.7 | 38        |
| 97  | Thermal and Stability Considerations of Super LWR during Sliding Pressure Startup. Journal of Nuclear Science and Technology, 2005, 42, 537-548.   | 0.7 | 23        |
| 98  | Evaluation of the Energy Conversion Ratio of Vapor Explosions for the Assessment of Nuclear Reactor Safety. Journal of Nuclear Science and Technology, 2005, 42, 28-39.                                | 0.7 | 3         |
| 99  | Thermal and Stability Considerations of Super LWR during Sliding Pressure Startup. Journal of Nuclear Science and Technology, 2005, 42, 537-548.   | 0.7 | 1         |
| 100 | Safety of Super LWR, (I) Safety System Design. Journal of Nuclear Science and Technology, 2005, 42, 927-934.   | 0.7 | 6         |
| 101 | Safety of Super LWR, (II) Safety Analysis at Supercritical Pressure. Journal of Nuclear Science and Technology, 2005, 42, 935-948.   | 0.7 | 10        |
| 102 | A Linear Stability Analysis of Supercritical Water Reactors, (II). Journal of Nuclear Science and Technology, 2004, 41, 1176-1186.   | 0.7 | 36        |
| 103 | Modelling of a single drop impact onto liquid film using particle method. International Journal for Numerical Methods in Fluids, 2004, 45, 1009-1023.  | 0.9 | 43        |
| 104 | A Linear Stability Analysis of Supercritical Water Reactors, (I). Journal of Nuclear Science and Technology, 2004, 41, 1166-1175.  | 0.7 | 63        |
| 105 | Startup Thermal Analysis of a High-Temperature Supercritical-Pressure Light Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 790-801.   | 0.7 | 19        |
| 106 | Numerical Simulation of Liquid Drop Deposition in Annular-Mist Flow Regime of Boiling Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 569-578.                                     | 0.7 | 32        |
| 107 | Numerical Analysis of Jet Breakup Behavior Using Particle Method. Journal of Nuclear Science and Technology, 2004, 41, 715-722.  | 0.7 | 49        |
| 108 | The Role of Fragmentation Mechanism in Large-Scale Vapor Explosions. JSME International Journal Series B, 2004, 47, 268-276.   | 0.3 | 0         |

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|-----|---|-----|-----------|
| 109 | A Linear Stability Analysis of Supercritical Water Reactors, (II) Coupled Neutronic Thermal-Hydraulic Stability. Journal of Nuclear Science and Technology, 2004, 41, 1176-1186.  | 0.7 | 2         |
| 110 | Numerical Analysis of Jet Breakup Behavior Using Particle Method. Journal of Nuclear Science and Technology, 2004, 41, 715-722.   | 0.7 | 8         |
| 111 | Startup Thermal Analysis of a High-Temperature Supercritical-Pressure Light Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 790-801.  | 0.7 | 5         |
| 112 | Three-dimensional numerical simulation of droplet deposition. The Proceedings of the JSME Annual Meeting, 2004, 2004.3, 207-208.  | 0.0 | 0         |
| 113 | Two-dimensional simulation of drop deformation and breakup at around the critical Weber number. Nuclear Engineering and Design, 2003, 225, 37-48.   | 0.8 | 63        |
| 114 | Numerical and Theoretical Investigation of Effect of Density Ratio on the Critical Weber Number of Droplet Breakup. Journal of Nuclear Science and Technology, 2003, 40, 501-508.   | 0.7 | 32        |
| 115 | Control of a High Temperature Supercritical Pressure Light Water Cooled and Moderated Reactor with Water Rods. Journal of Nuclear Science and Technology, 2003, 40, 298-306.  | 0.7 | 42        |
| 116 | Numerical Analysis of Pressure Propagation and Energy Conversion Ratio in Sodium Vapor Explosions. Nuclear Technology, 2003, 144, 324-336.  | 0.7 | 0         |
| 117 | Control of a High Temperature Supercritical Pressure Light Water Cooled and Moderated Reactor with Water Rods. Journal of Nuclear Science and Technology, 2003, 40, 298-306.  | 0.7 | 14        |
| 118 | Numerical and Theoretical Investigation of Effect of Density Ratio on the Critical Weber Number of Droplet Breakup. Journal of Nuclear Science and Technology, 2003, 40, 501-508.   | 0.7 | 8         |
| 119 | ICONE11-36047 The Role of Fragmentation Mechanism in Large-scale Vapor Explosions. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2003, 2003, 221.   | 0.0 | 1         |
| 120 | Investigation on Energetics of Ex-vessel Vapor Explosion Based on Spontaneous Nucleation Fragmentation. Journal of Nuclear Science and Technology, 2002, 39, 31-39.   | 0.7 | 2         |
| 121 | Light Water Cooled, High Temperature and High Performance Nuclear Power Plants Concept of Once-through Coolant Cycle, Supercritical-pressure, Light Water Cooled Nuclear Reactors. Nippon Genshiryoku Gakkaishi/Journal of the Atomic Energy Society of Japan, 2002, 44, 600-605. | 0.0 | 2         |
| 122 | Relationship between the structure of vapor explosion and fragmentation mechanisms. Nuclear Engineering and Design, 2002, 216, 121-137.   | 0.8 | 5         |
| 123 | Numerical Analysis of Heat Transfer Deterioration. The Reference Collection of Annual Meeting, 2002, VIII.02.1, 169-170.  | 0.0 | 0         |
| 124 | Coupled Analysis of Fluid and Structure Using Particle Method. The Reference Collection of Annual Meeting, 2002, VIII.02.1, 348-349.  | 0.0 | 0         |
| 125 | Conceptual Design of Once-through, Supercritical-pressure Light Water Cooled Nuclear Power Plants. The Reference Collection of Annual Meeting, 2002, VIII.02.1, 163-164.  | 0.0 | 1         |
| 126 | Numerical Analysis of Droplet Breakup Behavior using Particle Method. Journal of Nuclear Science and Technology, 2001, 38, 1057-1064.   | 0.7 | 140       |



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|-----|---|-----|-----------|
| 127 | Numerical Analysis of Jet Injection Behavior for Fuel-Coolant Interaction using Particle Method. Journal of Nuclear Science and Technology, 2001, 38, 174-182.  | 0.7 | 84        |
| 128 | Breeding Ratio Analysis of a Fast Reactor Cooled by Supercritical Light Water. Journal of Nuclear Science and Technology, 2001, 38, 703-710.  | 0.7 | 20        |
| 129 | Refinement of Transient Criteria and Safety Analysis for a High-Temperature Reactor Cooled by Supercritical Water. Nuclear Technology, 2001, 135, 252-264.  | 0.7 | 42        |
| 130 | Startup Thermal Considerations for Supercritical-Pressure Light Water-Cooled Reactors. Nuclear Technology, 2001, 134, 221-230.  | 0.7 | 17        |
| 131 | Direct calculation of bubble growth, departure, and rise in nucleate pool boiling. International Journal of Multiphase Flow, 2001, 27, 277-298.   | 1.6 | 144       |
| 132 | Supercritical-pressure, Once-through Cycle Light Water Cooled Reactor Concept. Journal of Nuclear Science and Technology, 2001, 38, 1081-1089.  | 0.7 | 79        |
| 133 | Numerical Analysis of Droplet Breakup Behavior using Particle Method.. Journal of Nuclear Science and Technology, 2001, 38, 1057-1064.  | 0.7 | 21        |
| 134 | Supercritical-pressure, Once-through Cycle Light Water Cooled Reactor Concept.. Journal of Nuclear Science and Technology, 2001, 38, 1081-1089.   | 0.7 | 8         |
| 135 | Numerical Analysis of Jet Injection Behavior for Fuel-Coolant Interaction using Particle Method.. Journal of Nuclear Science and Technology, 2001, 38, 174-182.   | 0.7 | 13        |
| 136 | Breeding Ratio Analysis of a Fast Reactor Cooled by Supercritical Light Water.. Journal of Nuclear Science and Technology, 2001, 38, 703-710.   | 0.7 | 5         |
| 137 | Subchannel analysis of a fast reactor cooled by supercritical light water. Progress in Nuclear Energy, 2000, 37, 197-204.   | 1.3 | 6         |
| 138 | Radiation Shielding for Fission Reactors. Journal of Nuclear Science and Technology, 2000, 37, 1-10.  | 0.7 | 6         |
| 139 | Numerical analysis of fragmentation mechanisms in vapor explosions. Nuclear Engineering and Design, 1999, 189, 423-433.   | 0.8 | 122       |
| 140 | Core design of a high-temperature fast reactor cooled by supercritical light water. Annals of Nuclear Energy, 1999, 26, 1423-1436.  | 0.9 | 29        |
| 141 | A particle-gridless hybrid method for incompressible flows. International Journal for Numerical Methods in Fluids, 1999, 30, 407-424.   | 0.9 | 63        |
| 142 | A Mesh-Free Numerical Method for Direct Simulation of Gas-Liquid Phase Interface. Nuclear Science and Engineering, 1999, 133, 192-200.  | 0.5 | 28        |
| 143 | Numerical Analysis of Sloshing with Large Deformation of Elastic Walls and Free Surfaces Using MPS method.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1999, 65, 2954-2960. | 0.2 | 7         |
| 144 | Numerical analysis of breaking waves using the moving particle semi-implicit method. International Journal for Numerical Methods in Fluids, 1998, 26, 751-769.  | 0.9 | 704       |

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|-----|--|-----|-----------|
| 145 | Measurement of decay heat of fast neutron fission products. Progress in Nuclear Energy, 1998, 32, 53-60.   | 1.3 | 1         |
| 146 | Study of epithermal neutron columns for boron neutron capture therapy. Progress in Nuclear Energy, 1998, 32, 61-70.  | 1.3 | 6         |
| 147 | Development of a LOCA analysis code for the supercritical-pressure light water cooled reactors. Annals of Nuclear Energy, 1998, 25, 1341-1361.   | 0.9 | 27        |
| 148 | Control of a Fast Reactor Cooled by Supercritical Light Water. Nuclear Technology, 1998, 121, 81-92.   | 0.7 | 28        |
| 149 | Pressure- and Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Supercritical-Pressure, Light-Water-Cooled Fast Reactor. Nuclear Technology, 1998, 123, 233-244.   | 0.7 | 13        |
| 150 | Numerical Analysis of Fragmentation Processes of Liquid Metal in Vapor Explosions Using Moving Particle Semi-implicit Method.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1998, 64, 2431-2437. | 0.2 | 7         |
| 151 | Numerical analysis of breaking waves using the moving particle semi-implicit method. International Journal for Numerical Methods in Fluids, 1998, 26, 751-769.   | 0.9 | 17        |
| 152 | Safety analysis of a supercritical pressure, light water cooled and moderated reactor with double tube water rods. Annals of Nuclear Energy, 1997, 24, 1447-1456.  | 0.9 | 7         |
| 153 | Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Light-Water-Cooled, Fast Breeder Reactor Operating at Supercritical Pressure. Journal of Nuclear Science and Technology, 1996, 33, 307-315.  | 0.7 | 20        |
| 154 | UO<sub>2</sub> Core Design of a Direct-Cycle Fast Converter Reactor Cooled by Supercritical Water. Nuclear Technology, 1996, 114, 273-284.   | 0.7 | 3         |
| 155 | Negative coolant void reactivity in large fast breeder reactors with hydrogenous moderator layer. Annals of Nuclear Energy, 1996, 23, 1105-1115.   | 0.9 | 38        |
| 156 | Core Design of a Direct-Cycle, Supercritical-Pressure, Light Water Reactor with Double Tube Water Rods. Journal of Nuclear Science and Technology, 1996, 33, 365-373.  | 0.7 | 19        |
| 157 | Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Light-Water-Cooled, Fast Breeder Reactor Operating at Supercritical Pressure.. Journal of Nuclear Science and Technology, 1996, 33, 307-315.   | 0.7 | 7         |
| 158 | Core Design of a Direct-Cycle, Supercritical-Pressure, Light Water Reactor with Double Tube Water Rods.. Journal of Nuclear Science and Technology, 1996, 33, 365-373.   | 0.7 | 7         |
| 159 | Systems Design of Direct-Cycle Supercritical-Water-Cooled Fast Reactors. Nuclear Technology, 1995, 109, 1-10.  | 0.7 | 34        |
| 160 | Design of water rod cores of a direct cycle supercritical-pressure light water reactor. Annals of Nuclear Energy, 1994, 21, 601-611.   | 0.9 | 34        |
| 161 | Numerical Analysis of Deterioration in Heat Transfer to Supercritical Water.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1994, 60, 2497-2503.  | 0.2 | 4         |
| 162 | A Direct-Cycle, Supercritical- Water-Cooled Fast Breeder Reactor. Journal of Nuclear Science and Technology, 1994, 31, 83-85.  | 0.7 | 9         |

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|-----|---|-----|-----------|
| 163 | Negative Void Reactivity in a Large Liquid-Metal Fast Breeder Reactor with Hydrogenous Moderator ( $ZrH_{1.7}$ ) Layers. Nuclear Technology, 1994, 107, 15-22.  | 0.7 | 5         |
| 164 | Core Design of a Direct-Cycle, Supercritical-Water-Cooled Fast Breeder Reactor. Nuclear Technology, 1994, 108, 24-32.   | 0.7 | 18        |
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