Yoshiaki Oka

List of Publications by Year
in descending order

222	4,482	117571	133188
papers	citations	34 h-index	59 g-index

228	228	228	1286
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Numerical analysis of breaking waves using the moving particle semi-implicit method. International Journal for Numerical Methods in Fluids, 1998, 26, 751-769.	0.9	704
2	Numerical investigation of heat transfer in upward flows of supercritical water in circular tubes and tight fuel rod bundles. Nuclear Engineering and Design, 2007, 237, 420-430.	0.8	153
3	Direct calculation of bubble growth, departure, and rise in nucleate pool boiling. International Journal of Multiphase Flow, 2001, 27, 277-298.	1.6	144
4	Numerical Analysis of Droplet Breakup Behavior using Particle Method. Journal of Nuclear Science and Technology, 2001, 38, 1057-1064.	0.7	140
5	Numerical analysis of fragmentation mechanisms in vapor explosions. Nuclear Engineering and Design, 1999, 189, 423-433.	0.8	122
6	A hybrid particle-mesh method for viscous, incompressible, multiphase flows. Journal of Computational Physics, 2005, 202, 65-93.	1.9	119
7	Numerical investigation on coalescence of bubble pairs rising in a stagnant liquid. Chemical Engineering Science, 2011, 66, 5055-5063.	1.9	104
8	Numerical Analysis of Jet Injection Behavior for Fuel-Coolant Interaction using Particle Method. Journal of Nuclear Science and Technology, 2001, 38, 174-182.	0.7	84
9	Supercritical-pressure, Once-through Cycle Light Water Cooled Reactor Concept. Journal of Nuclear Science and Technology, 2001, 38, 1081-1089.	0.7	79
10	Numerical computation of thermally controlled steam bubble condensation using Moving Particle Semi-implicit (MPS) method. Annals of Nuclear Energy, 2010, 37, 5-15.	0.9	73
11	A particle-gridless hybrid method for incompressible flows. International Journal for Numerical Methods in Fluids, 1999, 30, 407-424.	0.9	63
12	Two-dimensional simulation of drop deformation and breakup at around the critical Weber number. Nuclear Engineering and Design, 2003, 225, 37-48.	0.8	63
13	A Linear Stability Analysis of Supercritical Water Reactors, (I). Journal of Nuclear Science and Technology, 2004, 41, 1166-1175.	0.7	63
14	Concept and Design of a Supercritical-Pressure, Direct-Cycle Light Water Reactor. Nuclear Technology, 1993, 103, 295-302.	0.7	61
15	Three-dimensional Core Design of High Temperature Supercritical-Pressure Light Water Reactor with Neutronic and Thermal-Hydraulic Coupling. Journal of Nuclear Science and Technology, 2005, 42, 8-19.	0.7	60
16	Hamiltonian moving-particle semi-implicit (HMPS) method for incompressible fluid flows. Computer Methods in Applied Mechanics and Engineering, 2007, 196, 2876-2894.	3.4	60
17	Conceptual design of compact supercritical water-cooled fast reactor with thermal hydraulic coupling. Annals of Nuclear Energy, 2006, 33, 945-956.	0.9	57
18	Numerical investigation on bubble dynamics during flow boiling using moving particle semi-implicit method. Nuclear Engineering and Design, 2010, 240, 3830-3840.	0.8	55

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19	Ex-vessel molten core solidification behavior by moving particle semi-implicit method. Journal of Nuclear Science and Technology, 2012, 49, 1156-1164.	0.7	52
20	Numerical Analysis of Jet Breakup Behavior Using Particle Method. Journal of Nuclear Science and Technology, 2004, 41, 715-722.	0.7	49
21	Fuel, Core Design and Subchannel Analysis of a Superfast Reactor. Journal of Nuclear Science and Technology, 2008, 45, 138-148.	0.7	46
22	Numerical investigation of grid spacer effect on heat transfer of supercritical water flows in a tight rod bundle. International Journal of Thermal Sciences, 2014, 76, 245-257.	2.6	46
23	Modelling of a single drop impact onto liquid film using particle method. International Journal for Numerical Methods in Fluids, 2004, 45, 1009-1023.	0.9	43
24	Super Light Water Reactors and Super Fast Reactors. , 2010, , .		43
25	Refinement of Transient Criteria and Safety Analysis for a High-Temperature Reactor Cooled by Supercritical Water. Nuclear Technology, 2001, 135, 252-264.	0.7	42
26	Control of a High Temperature Supercritical Pressure Light Water Cooled and Moderated Reactor with Water Rods. Journal of Nuclear Science and Technology, 2003, 40, 298-306.	0.7	42
27	Fuel and Core Design of Super Light Water Reactor with Low Leakage Fuel Loading Pattern. Journal of Nuclear Science and Technology, 2006, 43, 129-139.	0.7	41
28	Review of R&D for supercritical water cooled reactors. Progress in Nuclear Energy, 2014, 77, 282-299.	1.3	41
29	Negative coolant void reactivity in large fast breeder reactors with hydrogenous moderator layer. Annals of Nuclear Energy, 1996, 23, 1105-1115.	0.9	38
30	Safety of Super LWR, (II). Journal of Nuclear Science and Technology, 2005, 42, 935-948.	0.7	38
31	Numerical simulation on void bubble dynamics using moving particle semi-implicit method. Nuclear Engineering and Design, 2009, 239, 2382-2390.	0.8	38
32	Experimental and numerical study of stratification and solidification/melting behaviors. Nuclear Engineering and Design, 2014, 272, 109-117.	0.8	38
33	A Linear Stability Analysis of Supercritical Water Reactors, (II). Journal of Nuclear Science and Technology, 2004, 41, 1176-1186.	0.7	36
34	Numerical investigation on melt freezing behavior in a tube by MPS method. Nuclear Engineering and Design, 2014, 273, 440-448.	0.8	36
35	Direct Cycle Light Water Reactor Operating at Supercritical Pressure. Journal of Nuclear Science and Technology, 1992, 29, 585-588.	0.7	35
36	Design of water rod cores of a direct cycle supercritical-pressure light water reactor. Annals of Nuclear Energy, 1994, 21, 601-611.	0.9	34

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37	Systems Design of Direct-Cycle Supercritical-Water-Cooled Fast Reactors. Nuclear Technology, 1995, 109, 1-10.	0.7	34
38	Numerical and Theoretical Investigation of Effect of Density Ratio on the Critical Weber Number of Droplet Breakup. Journal of Nuclear Science and Technology, 2003, 40, 501-508.	0.7	32
39	Numerical Simulation of Liquid Drop Deposition in Annular-Mist Flow Regime of Boiling Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 569-578.	0.7	32
40	Safety of Super LWR, (I) Safety System Design. Journal of Nuclear Science and Technology, 2005, 42, 927-934.	0.7	32
41	Experiments and MPS analysis of stratification behavior of two immiscible fluids. Nuclear Engineering and Design, 2013, 265, 210-221.	0.8	32
42	Numerical simulation of the SURC-2 and SURC-4 MCCI experiments by MPS method. Annals of Nuclear Energy, 2014, 73, 46-52.	0.9	31
43	Core design of a high-temperature fast reactor cooled by supercritical light water. Annals of Nuclear Energy, 1999, 26, 1423-1436.	0.9	29
44	Control of a Fast Reactor Cooled by Supercritical Light Water. Nuclear Technology, 1998, 121, 81-92.	0.7	28
45	A Mesh-Free Numerical Method for Direct Simulation of Gas-Liquid Phase Interface. Nuclear Science and Engineering, 1999, 133, 192-200.	0.5	28
46	Development of a LOCA analysis code for the supercritical-pressure light water cooled reactors. Annals of Nuclear Energy, 1998, 25, 1341-1361.	0.9	27
47	SAFETY OF THE SUPER LWR. Nuclear Engineering and Technology, 2007, 39, 257-272.	1.1	27
48	Direct simulation of flashing liquid jets using the MPS method. International Journal of Heat and Mass Transfer, 2006, 49, 402-405.	2.5	26
49	Numerical analysis of freezing controlled penetration behavior of the molten core debris in an instrument tube with MPS. Annals of Nuclear Energy, 2014, 71, 322-332.	0.9	26
50	Thermal and stability considerations for a supercritical water-cooled fast reactor with downward-flow channels during power-raising phase of plant startup. Nuclear Engineering and Design, 2009, 239, 665-679.	0.8	25
51	Core design for super fast reactor with all upward flow core cooling. Annals of Nuclear Energy, 2013, 57, 221-229.	0.9	25
52	Thermal and Stability Considerations of Super LWR during Sliding Pressure Startup. Journal of Nuclear Science and Technology, 2005, 42, 537-548.	0.7	23
53	Safety analysis of a supercritical-pressure water-cooled fast reactor under supercritical pressure. Nuclear Engineering and Design, 2010, 240, 1218-1228.	0.8	23
54	Molten uranium eutectic interaction on iron-alloy by MPS method. Nuclear Engineering and Design, 2014, 278, 387-394.	0.8	23

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55	Dynamic Analysis of Elastic Solids by MPS Method. Nihon Kikai Gakkai Ronbunshu, A Hen/Transactions of the Japan Society of Mechanical Engineers, Part A, 2005, 71, 16-22.	0.2	22
56	Subchannel analysis of supercritical light water-cooled fast reactor assembly. Nuclear Engineering and Design, 2007, 237, 1096-1105.	0.8	22
57	Numerical Analysis of Droplet Breakup Behavior using Particle Method.. Journal of Nuclear Science and Technology, 2001, 38, 1057-1064.	0.7	21
58	Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Light-Water-Cooled, Fast Breeder Reactor Operating at Supercritical Pressure. Journal of Nuclear Science and Technology, 1996, 33, 307-315.	0.7	20
59	Breeding Ratio Analysis of a Fast Reactor Cooled by Supercritical Light Water. Journal of Nuclear Science and Technology, 2001, 38, 703-710.	0.7	20
60	Fuel, Core Design and Subchannel Analysis of a Superfast Reactor. Journal of Nuclear Science and Technology, 2008, 45, 138-148.	0.7	20
61	Core Design of a Direct-Cycle, Supercritical-Pressure, Light Water Reactor with Double Tube Water Rods. Journal of Nuclear Science and Technology, 1996, 33, 365-373.	0.7	19
62	Startup Thermal Analysis of a High-Temperature Supercritical-Pressure Light Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 790-801.	0.7	19
63	3D simulation of eutectic interaction of Pbâ€“Sn system using Moving Particle Semi-implicit (MPS) method. Annals of Nuclear Energy, 2015, 81, 26-33.	0.9	19
64	Core Design of a Direct-Cycle, Supercritical-Water-Cooled Fast Breeder Reactor. Nuclear Technology, 1994, 108, 24-32.	0.7	18
65	Effect of Zirconium-Hydride Layers on Reducing Coolant Void Reactivity of Steam Cooled Fast Breeder Reactors. Journal of Nuclear Science and Technology, 1993, 30, 497-504.	0.7	17
66	Startup Thermal Considerations for Supercritical-Pressure Light Water-Cooled Reactors. Nuclear Technology, 2001, 134, 221-230.	0.7	17
67	Simulation of a Single Bubble Rising with Hybrid Particle-Mesh Method. Journal of Nuclear Science and Technology, 2007, 44, 886-893.	0.7	17
68	Numerical analysis of breaking waves using the moving particle semiâ€implicit method. International Journal for Numerical Methods in Fluids, 1998, 26, 751-769.	0.9	17
69	A cross-section sensitivity and uncertainty analysis of fusion reactor blankets with SAD/SED effect. Nuclear Engineering and Design/fusion: an International Journal Devoted To the Thermal, Mechanical, Materials, Structural, and Design Problems of Fusion Energy, 1986, 3, 287-300.	0.6	16
70	Simulation of drop deposition process in annular mist flow using three-dimensional particle method. Nuclear Engineering and Design, 2005, 235, 1687-1697.	0.8	16
71	Numerical analysis of the onset of droplet entrainment in annular two-phase flow by hybrid method. Annals of Nuclear Energy, 2010, 37, 230-240.	0.9	16
72	Plutonium breeding of light water cooled fast reactors. Journal of Nuclear Science and Technology, 2013, 50, 15-20.	0.7	16

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73	Sensitivity study of melt behavior of Fukushima Daiichi unit 1 type accident with MELCOR code and MPS method. Journal of Nuclear Science and Technology, 2015, 52, 109-121.	0.7	16
74	Measurements of Neutron and Gamma-Ray Streaming in a Cavity-Duct System and an Analysis by an Albedo Monte Carlo Method. Nuclear Science and Engineering, 1982, 81, 161-171.	0.5	15
75	LOCA Analysis of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 231-241.	0.7	15
76	Interpolating matrix method: A finite difference method for arbitrary arrangement of mesh points. Journal of Computational Physics, 1988, 75, 444-468.	1.9	14
77	Improvements of Feedwater Controller for the Super Fast Reactor. Journal of Nuclear Science and Technology, 2010, 47, 1155-1164.	0.7	14
78	Improved single pass core design for high temperature Super LWR. Nuclear Engineering and Design, 2014, 267, 100-108.	0.8	14
79	Control of a High Temperature Supercritical Pressure Light Water Cooled and Moderated Reactor with Water Rods. Journal of Nuclear Science and Technology, 2003, 40, 298-306.	0.7	14
80	A Design Study of the Neutron Irradiation Facility for Boron Neutron Capture Therapy. Nuclear Technology, 1981, 55, 642-655.	0.7	13
81	Pressure- and Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Supercritical-Pressure, Light-Water-Cooled Fast Reactor. Nuclear Technology, 1998, 123, 233-244.	0.7	13
82	Three-dimensional core analysis on a super fast reactor with negative local void reactivity. Nuclear Engineering and Design, 2009, 239, 408-417.	0.8	13
83	Safety analysis of a supercritical water cooled fast reactor with all-upward two-pass flow. Annals of Nuclear Energy, 2013, 59, 1-9.	0.9	13
84	Direct Cycle Light Water Reactor Operating at Supercritical Pressure.. Journal of Nuclear Science and Technology, 1992, 29, 585-588.	0.7	13
85	Numerical Analysis of Jet Injection Behavior for Fuel-Coolant Interaction using Particle Method.. Journal of Nuclear Science and Technology, 2001, 38, 174-182.	0.7	13
86	Fuel and Core Design of Super Light Water Reactor with Low Leakage Fuel Loading Pattern. , 0, .		13
87	Development of Statistical Thermal Design Procedure to Evaluate Engineering Uncertainty of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 32-42.	0.7	12
88	Single-pass core design of a low-temperature Super LWR. Journal of Nuclear Science and Technology, 2013, 50, 1129-1138.	0.7	12
89	ATWS Characteristics of Super LWR with/without Alternative Action. Journal of Nuclear Science and Technology, 2007, 44, 572-589.	0.7	11
90	CFD analysis of heat transfer in subchannels of a Super Fast Reactor. Nuclear Engineering and Design, 2010, 240, 1819-1829.	0.8	11

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91	Accidents and transients analyses of a super fast reactor with single flow pass core. Nuclear Engineering and Design, 2014, 273, 165-174.	0.8	11
92	Study on the LLFPs transmutation in a super-critical water-cooled fast reactor. Nuclear Engineering and Design, 2011, 241, 395-401.	0.8	10
93	Numerical investigation on practicability of reducing MCST by using grid spacer in a tight rod bundle. Nuclear Engineering and Design, 2014, 270, 198-208.	0.8	10
94	Single pass core design for a Super Fast Reactor. Annals of Nuclear Energy, 2015, 80, 451-459.	0.9	10
95	Safety of Super LWR, (II) Safety Analysis at Supercritical Pressure. Journal of Nuclear Science and Technology, 2005, 42, 935-948.	0.7	10
96	A Direct-Cycle, Supercritical- Water-Cooled Fast Breeder Reactor. Journal of Nuclear Science and Technology, 1994, 31, 83-85.	0.7	9
97	Thermo-mechanical analysis of supercritical pressure light water-cooled fast reactor fuel rod by FEMAXI-6 code. Annals of Nuclear Energy, 2006, 33, 1379-1390.	0.9	9
98	Experiments on Temperature and Flow Velocity Fluctuation of Coolant and the Possibility of Determination of Noise Source. Journal of Nuclear Science and Technology, 1972, 9, 330-338.	0.7	8
99	Core design of super LWR with double tube water rods. Nuclear Engineering and Design, 2014, 269, 340-348.	0.8	8
100	Supercritical-pressure, Once-through Cycle Light Water Cooled Reactor Concept.. Journal of Nuclear Science and Technology, 2001, 38, 1081-1089.	0.7	8
101	Numerical and Theoretical Investigation of Effect of Density Ratio on the Critical Weber Number of Droplet Breakup. Journal of Nuclear Science and Technology, 2003, 40, 501-508.	0.7	8
102	Numerical Analysis of Jet Breakup Behavior Using Particle Method. Journal of Nuclear Science and Technology, 2004, 41, 715-722.	0.7	8
103	Experiments and Analyses of Neutron and Gamma-Ray Streaming in the Cavity-Duct System of a Fast Neutron Source Reactor. Nuclear Science and Engineering, 1980, 76, 119-136.	0.5	7
104	Neutron Streaming through a Slit and Duct in Concrete Shields and Comparison with a Monte Carlo Analysis. Nuclear Science and Engineering, 1983, 84, 337-344.	0.5	7
105	Safety analysis of a supercritical pressure, light water cooled and moderated reactor with double tube water rods. Annals of Nuclear Energy, 1997, 24, 1447-1456.	0.9	7
106	Numerical Analysis of Fragmentation Processes of Liquid Metal in Vapor Explosions Using Moving Particle Semi-implicit Method.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1998, 64, 2431-2437.	0.2	7
107	Numerical Analysis of Sloshing with Large Deformation of Elastic Walls and Free Surfaces Using MPS method.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1999, 65, 2954-2960.	0.2	7
108	Passive safety system of a super fast reactor. Nuclear Engineering and Design, 2015, 289, 117-125.	0.8	7

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109	Effect of Zirconium-Hydride Layers on Reducing Coolant Void Reactivity of Steam Cooled Fast Breeder Reactors.. Journal of Nuclear Science and Technology, 1993, 30, 497-504.	0.7	7
110	Flow-Induced Accident and Transient Analyses of a Direct-Cycle, Light-Water-Cooled, Fast Breeder Reactor Operating at Supercritical Pressure.. Journal of Nuclear Science and Technology, 1996, 33, 307-315.	0.7	7
111	Core Design of a Direct-Cycle, Supercritical-Pressure, Light Water Reactor with Double Tube Water Rods.. Journal of Nuclear Science and Technology, 1996, 33, 365-373.	0.7	7
112	Numerical Simulation of Liquid Drop Deposition in Annular-Mist Flow Regime of Boiling Water Reactor. , 0, .		7
113	A conceptual design of light ion beam fusion reactorâ€"UTLIF(1). Nuclear Engineering and Design, 1983, 74, 377-392.	0.8	6
114	Design and safety study of power producing fusion-fission hybrid reactor. Nuclear Engineering and Design/fusion: an International Journal Devoted To the Thermal, Mechanical, Materials, Structural, and Design Problems of Fusion Energy, 1984, 1, 255-263.	0.6	6
115	Study of epithermal neutron columns for boron neutron capture therapy. Progress in Nuclear Energy, 1998, 32, 61-70.	1.3	6
116	Subchannel analysis of a fast reactor cooled by supercritical light water. Progress in Nuclear Energy, 2000, 37, 197-204.	1.3	6
117	Radiation Shielding for Fission Reactors. Journal of Nuclear Science and Technology, 2000, 37, 1-10.	0.7	6
118	Numerical Analyses of Flashing Jet Structure and Droplet Size Characteristics. Journal of Nuclear Science and Technology, 2006, 43, 285-294.	0.7	6
119	CFD analyses in tight-lattice subchannels and seven-rod bundle geometries of a Super Fast Reactor. Nuclear Engineering and Design, 2011, 241, 1656-1666.	0.8	6
120	Time dependent start-up thermal analysis of a Super Fast Reactor. Nuclear Engineering and Design, 2013, 263, 129-137.	0.8	6
121	Nuclear Reactor Calculations. An Advanced Course in Nuclear Engineering, 2014, , 49-126.	0.1	6
122	Neutronic Feasibility of Supercritical Steam Cooled Fast Breeder Reactor.. Journal of Nuclear Science and Technology, 1991, 28, 585-587.	0.7	6
123	Safety of Super LWR, (I) Safety System Design. Journal of Nuclear Science and Technology, 2005, 42, 927-934.	0.7	6
124	Neutron and Gamma-Ray Penetrations in Thick Iron. Nuclear Science and Engineering, 1980, 73, 259-273.	0.5	5
125	Development Studies Regarding the Construction of Epithermal-Enriched Neutron Field for Medical Purposes at the University of Tokyo YAYOI Fast Reactor. Nuclear Technology, 1980, 48, 204-215.	0.7	5
126	A conceptual design of light ion beam fusion reactor â€" UTLIF(2). Nuclear Engineering and Design/fusion: an International Journal Devoted To the Thermal, Mechanical, Materials, Structural, and Design Problems of Fusion Energy, 1984, 1, 387-407.	0.6	5

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127	Development of Three-Dimensional Space Dependent Kinetics Code IBIS and Analysis of Transient Behavior of LMFBR Cores. Journal of Nuclear Science and Technology, 1985, 22, 788-801.	0.7	5
128	Measurement and Analysis of Neutron Leakage Spectrum from Nickel Sphere for 14 MeV Neutron Source. Journal of Nuclear Science and Technology, 1986, 23, 477-486.	0.7	5
129	Development of Radiation Transport Code in Three-Dimensional (X, Y, Z)Geometry for Shielding Analyses by Direct Integration Method. Journal of Nuclear Science and Technology, 1987, 24, 181-193.	0.7	5
130	Accuracy of Multi-Group Transport Calculation in D-T Fusion Neutronics. Journal of Nuclear Science and Technology, 1987, 24, 333-339.	0.7	5
131	Negative Void Reactivity in a Large Liquid-Metal Fast Breeder Reactor with Hydrogenous Moderator (ZrH_{1.7}) Layers. Nuclear Technology, 1994, 107, 15-22.	0.7	5
132	Relationship between the structure of vapor explosion and fragmentation mechanisms. Nuclear Engineering and Design, 2002, 216, 121-137.	0.8	5
133	Total loss of flow accident characteristics of Super FR with new coolant flow. Nuclear Engineering and Design, 2013, 257, 155-160.	0.8	5
134	High Breeding Core of a Supercritical-Pressure Light Water Cooled Fast Reactor. , 2013, , .		5
135	Concept and nuclear performance of power-producing fast fission blankets for fusion-fission hybrid reactors using equilibrium fissile fuel.. Journal of Nuclear Science and Technology, 1982, 19, 166-168.	0.7	5
136	Simulation of multigroup X-ray, alpha-particle and neutron transport in ion beam driven ICF target.. Journal of Nuclear Science and Technology, 1985, 22, 683-696.	0.7	5
137	Breeding Ratio Analysis of a Fast Reactor Cooled by Supercritical Light Water.. Journal of Nuclear Science and Technology, 2001, 38, 703-710.	0.7	5
138	Startup Thermal Analysis of a High-Temperature Supercritical-Pressure Light Water Reactor. Journal of Nuclear Science and Technology, 2004, 41, 790-801.	0.7	5
139	Risks and benefits of evacuation in TEPCO's Fukushima Daiichi nuclear power station accident. Progress in Nuclear Energy, 2022, 148, 104222.	1.3	5
140	Concept and Nuclear Performance of Power-Producing Fast Fission Blankets for Fusion-Fission Hybrid Reactors Using Equilibrium Fissile Fuel. Journal of Nuclear Science and Technology, 1982, 19, 166-168.	0.7	4
141	Simulation of Multigroup X-Ray, Alpha-Particle and Neutron Transport in Ion Beam Driven ICF Target. Journal of Nuclear Science and Technology, 1985, 22, 683-696.	0.7	4
142	Development of Method Based on Perturbation Theory for Estimating and Correcting Errors in Numerical Solution of Transport Equation. Journal of Nuclear Science and Technology, 1987, 24, 173-180.	0.7	4
143	Neutronic Feasibility of Supercritical Steam Cooled Fast Breeder Reactor. Journal of Nuclear Science and Technology, 1991, 28, 585-587.	0.7	4
144	Numerical Analysis of Deterioration in Heat Transfer to Supercritical Water.. 880-02 Nihon Kikai Gakkai RonbunshÅ« Transactions of the Japan Society of Mechanical Engineers Series B B-hen, 1994, 60, 2497-2503.	0.2	4

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145	Numerical Solution on Spherical Vacuum Bubble Collapse Using MPS Method. Journal of Engineering for Gas Turbines and Power, 2010, 132, .	0.5	4
146	Accuracy of nuclear design of fast and thermal neutron coupled core by SRAC. Progress in Nuclear Energy, 2014, 71, 82-88.	1.3	4
147	Improvements of two-pass core design for super fast reactor. Annals of Nuclear Energy, 2014, 69, 108-115.	0.9	4
148	Safety analysis of a Super LWR with double tube water rods. Nuclear Engineering and Design, 2014, 266, 129-136.	0.8	4
149	Subchannel analysis with turbulent mixing rate of supercritical pressure fluid. Nuclear Engineering and Design, 2015, 287, 119-130.	0.8	4
150	CFD analysis of coolant channel geometries for a tightly packed fuel rods assembly of Super FBR. Nuclear Engineering and Design, 2015, 288, 119-129.	0.8	4
151	Analysis of anticipated transient without scram of a Super Fast Reactor with single flow pass core. Annals of Nuclear Energy, 2015, 75, 54-63.	0.9	4
152	Development of method based on perturbation theory for estimating and correcting errors in numerical solution of transport equation.. Journal of Nuclear Science and Technology, 1987, 24, 173-180.	0.7	4
153	Development of Statistical Thermal Design Procedure to Evaluate Engineering Uncertainty of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 32-42.	0.7	4
154	The Effect of Coolant Orificing on the Core Performance of a Heterogeneous Liquid-Metal Fast Breeder Reactor. Nuclear Technology, 1983, 61, 7-16.	0.7	3
155	Fusion nuclear systems design and analysis. Fusion Engineering and Design, 1988, 7, 369-376.	1.0	3
156	Boundary-fitted coordinate method for incompressible flow using Riemann geometry.. Nippon Genshiryoku Gakkaishi/Journal of the Atomic Energy Society of Japan, 1990, 32, 819-833.	0.0	3
157	UO₂Core Design of a Direct-Cycle Fast Converter Reactor Cooled by Supercritical Water. Nuclear Technology, 1996, 114, 273-284.	0.7	3
158	Evaluation of the Energy Conversion Ratio of Vapor Explosions for the Assessment of Nuclear Reactor Safety. Journal of Nuclear Science and Technology, 2005, 42, 28-39.	0.7	3
159	Measurements of Neutron Capture Cross Section of²³⁷Np for Fast Neutrons. Journal of Nuclear Science and Technology, 2009, 46, 460-468.	0.7	3
160	CFD analysis of coolant channel geometries for a tightly packed fuel rods assembly at subcritical pressure. Nuclear Engineering and Design, 2015, 284, 115-129.	0.8	3
161	Some Relations of Neutronic Noise with Fluctuation of Inlet Coolant Temperature and Vibration of a Control Rod Obtained by Simultaneous Measurements at KUR. Journal of Nuclear Science and Technology, 1977, 14, 869-877.	0.7	3
162	Concept and nuclear performance of direct-enrichment fusion breeder blanket using UO2 powder.. Journal of Nuclear Science and Technology, 1985, 22, 165-173.	0.7	3

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163	Development of three-dimensional space dependent kinetics code IBIS and analysis of transient behavior of LMFBR cores.. Journal of Nuclear Science and Technology, 1985, 22, 788-801.	0.7	3
164	Development of radiation transport code in three-dimensional (X, Y, Z) geometry for shielding analyses by direct integration method.. Journal of Nuclear Science and Technology, 1987, 24, 181-193.	0.7	3
165	Accuracy of multi-group transport calculation in D-T fusion neutronics.. Journal of Nuclear Science and Technology, 1987, 24, 333-339.	0.7	3
166	A Linear Stability Analysis of Supercritical Water Reactors, (I). , 0, .		3
167	LOCA Analysis of Super LWR. Journal of Nuclear Science and Technology, 2006, 43, 231-241.	0.7	3
168	Numerical Analyses of Flashing Jet Structure and Droplet Size Characteristics. Journal of Nuclear Science and Technology, 2006, 43, 285-294.	0.7	3
169	ATWS Characteristics of Super LWR with/without Alternative Action. Journal of Nuclear Science and Technology, 2007, 44, 572-589.	0.7	3
170	Cell Flowmeter for Turbulent Velocity Fluctuation Measurements in Water. Journal of Nuclear Science and Technology, 1972, 9, 253-260.	0.7	2
171	Measurement of neutron noise in the presence of a vibrating control plate. Annals of Nuclear Energy, 1977, 4, 127-133.	0.9	2
172	Concept and Nuclear Performance of Direct-Enrichment Fusion Breeder Blanket Using UO₂Powder. Journal of Nuclear Science and Technology, 1985, 22, 165-173.	0.7	2
173	Investigation on Energetics of Ex-vessel Vapor Explosion Based on Spontaneous Nucleation Fragmentation. Journal of Nuclear Science and Technology, 2002, 39, 31-39.	0.7	2
174	Light Water Cooled, High Temperature and High Performance Nuclear Power Plants Concept of Once-through Coolant Cycle, Supercritical-pressure, Light Water Cooled Nuclear Reactors. Nippon Genshiryoku Gakkaishi/Journal of the Atomic Energy Society of Japan, 2002, 44, 600-605.	0.0	2
175	Fuel rod behavior under normal operating conditions in Super Fast Reactor with high power density. Nuclear Engineering and Design, 2015, 289, 166-174.	0.8	2
176	A Linear Stability Analysis of Supercritical Water Reactors, (II) Coupled Neutronic Thermal-Hydraulic Stability. Journal of Nuclear Science and Technology, 2004, 41, 1176-1186.	0.7	2
177	Simulation of a Single Bubble Rising with Hybrid Particle-Mesh Method. Journal of Nuclear Science and Technology, 2007, 44, 886-893.	0.7	2
178	Experimental Study on Fast-Neutron Streaming through a Grid-Plate Shield of a Liquid-Metal Fast Breeder Reactor. Nuclear Technology, 1976, 31, 287-296.	0.7	1
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